



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 30 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1. INTRODUCTION

On September 24, 1987, Rochester Gas and Electric Corporation submitted an application for amending the Technical Specification to incorporate the requirements for the Reactor Vessel Level Indication System (RVLIS). Action on this proposed revision was delayed pending the final approval of RVLIS by the NRC staff. On April 26, 1988, the staff completed the review of RVLIS of GINNA and found it acceptable, provided the licensee committed to comply with certain staff implementation requirements. The licensee complied with such requirements in their letter of May 3, 1988.

2. EVALUATION

In March 1986, the licensee installed the RVLIS to satisfy a NUREG-0737 requirement. The system supplements existing instrumentation in order to insure an unambiguous, easy-to-interpret indication of inadequate reactor core cooling. The proposed Technical Specifications are in accordance with NRC guidelines in that they meet the required number of instrument channels, the frequency for checks, calibration and testing of the system. RVLIS does not actuate any control function of the reactor system. Its purpose is to provide the plant operator additional information on reactor vessel water level, particularly during transient events. Such data is utilized in the GINNA emergency operating procedures. The proposed Technical Specifications on RVLIS do not create any new or different kind of accident because they do not interact with any safety aspects of plant operations. The additional information, provided to the operator, may increase margins of safety for plant operators during unusual events. Early indication on whether or not the water level is responding to corrective measures taken during an event is important information for safe operations.

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3. ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4. CONCLUSION

The staff evaluated the licensee's request to revise the Technical Specifications to incorporate the Reactor Vessel Level Indication System. The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 23, 1988

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