



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R.E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 30
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated September 24, 1987 as supplemented on May 3, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

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PDR ADOCK 05000244
P PNU

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 30, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 23, 1988

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 30, 1988

OFC	: PD#1-3	: PD#1-3	: <i>OGC-Book</i>	: DIRPD#1-3	:	:	:
NAME	: <i>MRushbrook</i>	: <i>CS</i>	: <i>Bachmann</i>	: <i>Wessman</i>	:	:	:
DATE	: <i>9/29/88</i>	: <i>9/19/88</i>	: <i>9/15/88</i>	: <i>9/20/88</i>	:	:	:

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ATTACHMENT TO LICENSE AMENDMENT NO.30

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A as follows:

REMOVE

page 3.5-16
page 4.1-7

INSERT

page 3.5-16
page 4.1-7

TABLE 3.5-3
Accident Monitoring Instrumentation

	TOTAL REQUIRED NO. OF CHANNELS (7)	MINIMUM CHANNELS OPERABLE (7)
1. Pressurizer Water Level (1)	2	1
2. Auxiliary Feedwater Flow Rate (2)(3)	2/steam generator	1/steam generator
3. Steam Generator Water Level - Wide Range (3)	1/steam generator	1/steam generator
4. Reactor Coolant System Subcooling Margin Monitor (4)	2	1
5. Pressurizer PORV Position Indicator (5)	2/Valve	1/Valve
6. PORV Block Valve Position Indicator (1)	1/Valve	0/Valve
7. Pressurizer Safety Valve Position Indicator (5)	2/Valve	1/Valve
8. Containment Pressure (8)	2	1
9. Containment Water Level (Narrow Range, Sump A)	1(6)	1(6)
10. Containment Water Level (Wide Range, Sump B)	2	1
11. Core-Exit Thermocouples	4/core quadrant	2/core quadrant
12. Reactor Vessel Level Indication System	2	1

Notes

- (1) Emergency power for pressurizer equipment, NUREG-0737, item II.G.1.
- (2) Auxiliary feedwater system flow indication, NUREG-0737, item II.E.1.2.
- (3) Only 2 out of the 3 indications (two steam generator auxiliary feedwater flow and one wide-range steam generator level) are required to be operable, NUREG-0737, item II.E.1.2.
- (4) Instrumentation for detection of inadequate core cooling, NUREG-0737, item II.F.2.1.
- (5) Direct indication of relief and safety valve position, NUREG-0737, item II.D.3. Two channels include a primary detector and RTD as the backup detector.
- (6) Operation may continue with less than the minimum channels operable provided that the requirements of Technical Specification 3.1.5.1 are met.
- (7) See Specification 3.5.3 for required action.
- (8) Containment pressure monitor, NUREG-0737, item II.F.1.4.

TABLE 4.1-1 (CONTINUED)

	<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
25.	Containment Pressure	S	R	M	Narrow range containment pressure (-3.0, +3 psig) excluded
26.	Steam Generator Pressure	S	R	M	
27.	Turbine First Stage Pressure	S	R	M	
28.	Emergency Plan Radiation Instruments	M	R	M	
29.	Environmental Monitors	M	NA	NA	
30.	Loss of Voltage/Degraded Voltage 480 Volt Safeguards Bus	NA	R	M	
31.	Trip of Main Feedwater Pumps	NA	NA	R	
32.	Steam Flow	S	R	M	
33.	T _{AVG}	S	R	M	
34.	Chlorine Detector, Control Room Air Intake	NA	R	M	
35.	Ammonia Detector, Control Room Air Intake	NA	R	M	
36.	Radiation Detectors, Control Room Air Intake	NA	R	M	
37.	Reactor Vessel Level Indication System	M	R	NA	