

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING AMENDMENT NO. 27 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

INTRODUCTION

1.0 On September 23, 1987, the Rochester Gas and Electric Corporation requested an amendment to the Technical Specifications to change the reporting requirements for primary coolant iodine spiking and the elimination of the 800 hour shutdown requirement.

2.0 EVALUATION

Revised guidance as to the reporting and 800 hour shutdown requirements was provided in Generic Letter 85-19, dated September 27, 1985, "Reporting Requirements on Primary Coolant Iodine Spikes". The letter stated that (1) the reporting of primary coolant iodine spiking activity levels can be reduced from a short-term report (i.e., Special Report or Licensee Event Report) to an item to be evaluated in the Annual Report and (2) existing shutdown requirements based on exceeding the primary coolant specific activity limits for an accumulated period of over 800 hours were no longer necessary. The change in these requirements is based on an improvement in the quality of nuclear fuel over the past 10 years; and the fact that appropriate actions would be initiated long before approaching the limit specified in the Technical Specification. An example is that a Technical Specification provision requires the reactor to be subcritical within 8 hours if the primary coolant iodine level exceeds a certain value after 168 hours of continous operation. Also, Generic Letter 85-19 provided guidance on the reporting of information. The proposed change to the Technical Specification on reporting is revised in accordance with the Generic Letter. The staff finds the revisions to be acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to use of a facility component located within the restricted area as deferred in 10 CFR Part 20. This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding.

This amendment also involves a change in the recordkeeping, reporting, or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

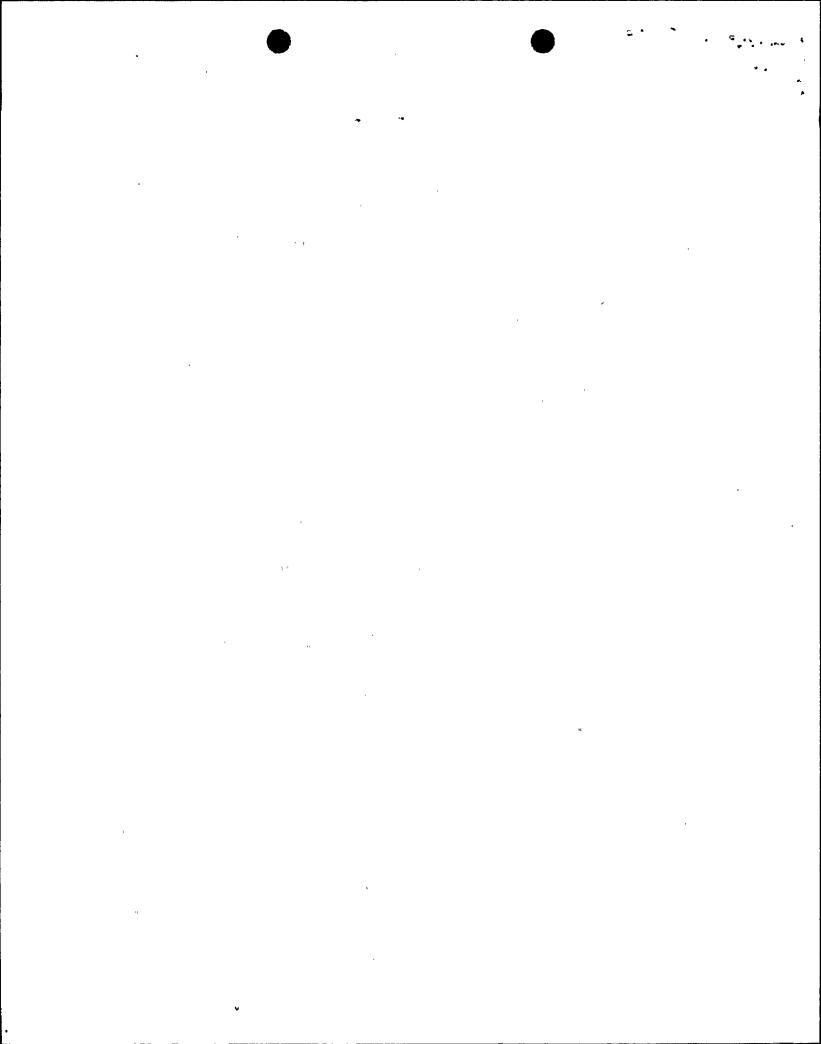
The staff has evaluated the licensee's request to change the reporting requirements related to primary coolant specific activity levels, specifically the reporting of primary coolant iodine spikes can be reduced from a short-term report to an annual report. The Annual Report requirements are specified to more clearly designate the results to be included from the specific activity analysis than was previously required.

The staff has concluded, based on the considerations discussed above, that; (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations; and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 Acknowledgement

Prinicipal Contributors: Carl Stahle

Dated: May 31, 1988





UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

March 28, 1988

Docket No. 50-244

Mr. Roger Kober, Vice President Electric and Steam Production Rochester Gas & Electric Corporation 89 East Avenue Rochester, New York 14649

Dear Mr. Kober:

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SUBJECT: AMENDMENT NO. 26 TO FACILITY OPERATING LICENSE NO. DPR-18

(TAC NO. 65424)

The Commission has issued the enclosed Amendment No.26 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment consists of changes to the license in response to your submittals dated November 21, 1986 and August 18, 1987.

This amendment modifies paragraph 2.E of the license to require compliance with the amended Physical Security Plan. This Plan was amended to conform to the requirements of 10 CFR 73.55. Consistent with the provisions of 10 CFR 73.55, search requirements must be implemented within 60 days and miscellaneous amendments within 180 days from the effective date of this amendment.

Our evaluation of the amendment to your Physical Security Plan for the R. E. Ginna Nuclear Power Plant is contained in the enclosed Safeguards Evaluation Report. Based on this evaluation, we find that you meet the requirements of the Miscellaneous Amendments and Search Requirements revision to 10 CFR 73.55 and the recordkeeping requirements of 10 CFR 73.70.

Your Physical Security Plan consists of Safeguards Information required to be protected from unauthorized disclosure in accordance with the provisions of 10 CFR 73.21.

Notice of Issuance will be included in the Commission's biweekly <u>Federal</u> Register Notice.

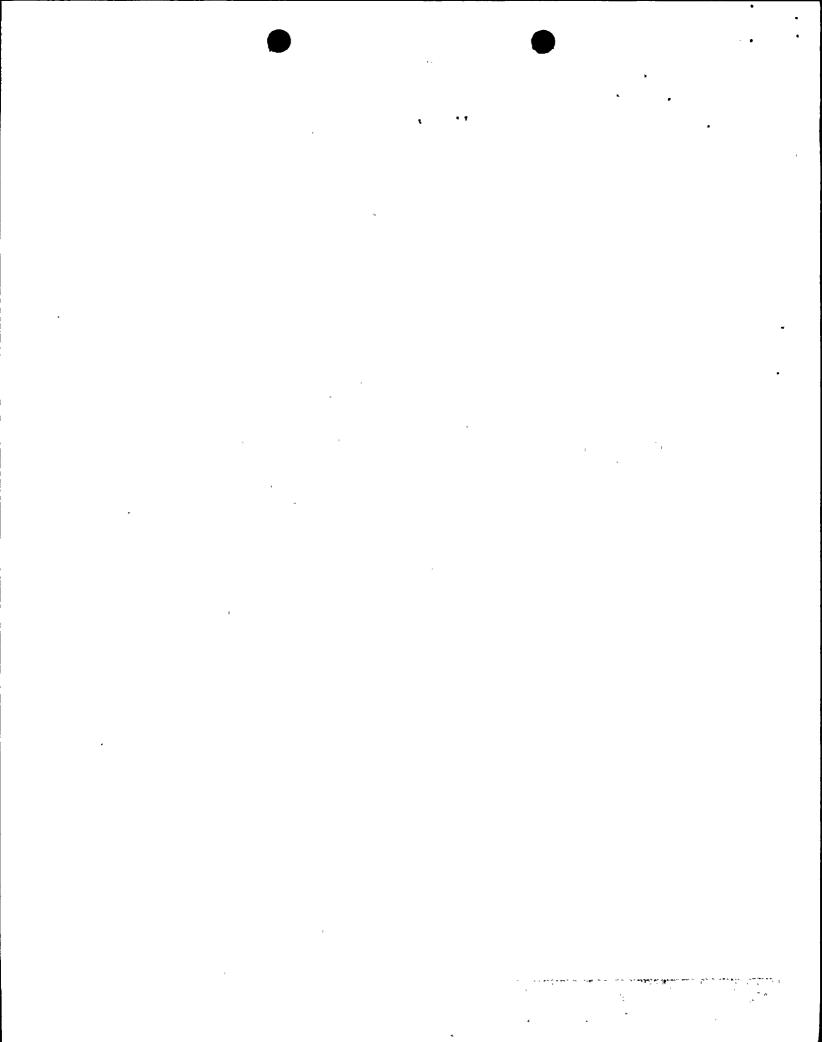
Sincerely, Carl Stahle, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:

 Amendment No. 26 to License No. DPR-18

2. Safeguards Evaluation Report

cc w/enclosures:
See next page



Mr. Roger W. Kober Rochester Gas and Electric Corporation Nuclear Power Plant

R. E. Ginna

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Sincerely,

Carl Stahle, Project Manager Project Directorate I-3
Division of Reactor Projects I/II

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"Robert Emmet Ginna Nuclear Plant Physical Security Plan," with revisions submitted through August 18, 1987; "Robert Emmet Ginna Nuclear Plant Guard Training and Qualification Plan" with revisions submitted through July 30, 1981; and "Robert Emmet Ginna Nuclear Plant Safeguards Contingency Plan" with revisions submitted through April 14, 1981. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard H. Wessman, Director Project Directorate I-3 Division of Reactor Projects I/II

Date of Issuance: March 28, 1988

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