



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION
DOCKET NO. 50-244
R. E. GINNA NUCLEAR POWER PLANT
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 27
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated September 23, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 27, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 31, 1988

ATTACHMENT TO LICENSE AMENDMENT NO.27

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.1-21
3.1-22
6.9-6
6.9-7

INSERT

3.1-21
3.1-22*
6.9-6
6.9-7

*No change - repositioned on page

3.1.4 Maximum Coolant Activity

Specifications

3.1.4.1 Whenever the reactor is critical or the reactor coolant average temperature is greater than 500°F:

- a. The total specific activity of the reactor coolant shall not exceed $84/\bar{E}$ $\mu\text{Ci/gm}$, where \bar{E} is the average beta and gamma energies per disintegration in Mev.
- b. The I-131 equivalent of the iodine activity in the reactor coolant shall not exceed 0.2 $\mu\text{Ci/gm}$.
- c. The I-131 equivalent of the iodine activity on the secondary side of a steam generator shall not exceed 0.1 $\mu\text{Ci/gm}$.

3.1.4.2 If the limit of 3.1.4.1.a is exceeded, then be subcritical with reactor coolant average temperature less than 500°F within 8 hours.

3.1.4.3 . a. If the I-131 equivalent activity in the reactor coolant exceeds the limit of 3.1.4.1.b but is less than the allowable limit shown on Figure 3.1.4-1, operation may continue for up to 168 hours. If the I-131 equivalent activity in the reactor coolant exceeds the limit of 3.1.4.1.b for more than 500 hours in any consecutive 6-month period, then prepare and submit a report to the Commission pursuant to Specification 6.9.2.

The reactor may be taken critical or reactor coolant average temperature may be increased above a 500°F with the I-131 equivalent activity greater than the limit of 3.1.4.1.b as long as the provisions of this paragraph are met.

- b. If the I-131 equivalent activity exceeds the limit of 3.1.4.1.b for more than 168 hours during one continuous time interval or exceeds the limit shown on Figure 3.1.4-1, be subcritical with reactor coolant average temperature less than 500°F within 8 hours.
- c. If the I-131 equivalent activity exceeds the limit of 3.1.4.1.b, then perform sampling and analysis as required by Table 4.1-4, item 4a, until the activity is reduced to less than the limit of 3.1.4.1.b.

3.1.4.4 If the limit of 3.1.4.1.c is exceeded, then be at hot shutdown within 8 hours and in cold shutdown within the following 32 hours.

Basis:

The total activity limit for the primary system corresponds to operation with the plant design basis of 1% fuel defects.⁽¹⁾

Radiation shielding and the radioactive waste disposal systems

6.9.2 Unique Reporting Requirements

6.9.2.1 Annually: Results of required leak test performed on sources if the tests reveal the presence of 0.005 microcurie or more of removable contamination.

6.9.2.2 Annually: A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man-rem exposure according to work and job functions, e.g., reactor operations and surveillance, in-service inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions. (NOTE: This tabulation supplements the requirements of Section 20.407 of 10CFR Part 20.)

6.9.2.3 Annually: The results of specific activity analysis in which the primary coolant exceeded the limits of Specification 3.1.4.1.a and b. The following information shall be included: (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded; (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit,

results of analyses while the limit was exceeded and results of one analysis after the radioiodine activity was reduced to less than the limit. Each result should include the date and time of sampling and the radioiodine concentrations; (3) Clean-up system flow history starting 48 hours prior to the first sample in which the limit was exceeded; (4) Graph of the I-131 concentration and one other radioiodine isotope concentration as a function of time for the duration of the specific activity above the steady-state level; and (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.

6.9.2.4 Reactor Overpressure Protection System Operation

In the event either the PORVs or the RCS vent(s) are used to mitigate a RCS pressure transient, a Special Report shall be prepared and submitted to the Commission within thirty days. The report shall describe the circumstances initiating the transient, the effect of the PORVs or vent(s) on the transient and any other corrective action necessary to prevent recurrence.

6.9.2.5 Special reports shall be submitted to the Director of the NRC Regional Office listed in Appendix D, 10CFR Part 20, with a copy to the Director, Office of Inspection and Enforcement, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555 within the time period specified for each report.