

APPENDIX A

NOTICE OF VIOLATION

Rochester Gas and Electric Corporation
R. E. Ginna Nuclear Power Plant

Docket No. 50-244
License No. DPR-18

As a result of the inspection conducted on March 13-17, 1989, and in accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (Enforcement Policy), the following violation was identified:

10 CFR 50, Appendix B, Criterion III requires that measures be established to assure that applicable regulatory requirements.....are correctly translated into specifications, drawings, procedures and instructions." It further requires that, "design changes, including field changes shall be subject to design control measures commensurate with the original design."

Contrary to the above, the licensee's design criteria and safety analysis for the PORV block valve replacement to be performed during the 1989 refueling outage did not address operability qualification of the replacement valves. Operability qualification of PORV block valves was a NUREG 0737 Item (II.D.1) committed to by the licensee as confirmed in Commission Order dated July 10, 1981.

This is a Severity Level IV Violation (Supplement I)

Pursuant to 10 CFR 2.201, Rochester Gas and Electric Corporation is hereby required to submit to this office, within 30 days of the date of the letter transmitting this Notice, a written statement or explanation in reply, including: (1) The reasons for the violations; (2) the corrective steps which have been taken and the results achieved; (3) corrective steps which will be taken to avoid further violations; and (4) the date when full compliance will be achieved. Where good cause is shown, consideration will be given to extending this response time.

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