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 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
 AUTH. NAME AUTHOR AFFILIATION
 MECREDDY, R.C. Rochester Gas & Electric Corp.
 RECIPIENT NAME RECIPIENT AFFILIATION
 JOHNSON, A.R. Project Directorate I-3

SUBJECT: Provides info re status of USI requirements, per Generic Ltr 89-21.

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 TITLE: Generic Ltr 89-21 Response, Implementation of Unresolved Safety Issue

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November 27, 1989

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U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Allen R. Johnson
Project Directorate I-3
Washington, D.C. 20555

Subject: Response to Generic Letter 89-21
Status of Unresolved Safety Issue Requirements
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Johnson:

By Generic Letter 89-21 from James G. Partlow, dated October 19, 1989, you requested information regarding the status of unresolved safety issue requirements. The attachment to the October 19 letter has been marked to show the status as requested and is enclosed with this letter.

Very truly yours,


Robert C. Mecredy
General Manager
Nuclear Production

JRJ\069
Enclosure

xc: Mr. Allen R. Johnson (Mail Stop 14D1)
Project Directorate I-3
Washington, D.C. 20555

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Ginna Senior Resident Inspector

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UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	All	C/Mar 1984	NUREG-0927 SER Dec 20, 1979
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	C/Sept 1986	NUREG-0609 GL 84-04 NRC Ltr Sept 9, 1986
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	C/Sept 1988	NUREG-0844 T.S. Amendment #35 Apr 24, 1989
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	N/A	
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	N/A	
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	N/A	

* C - COMPLETE
 NC - NO CHANGES NECESSARY
 NA - NOT APPLICABLE
 I - INCOMPLETE
 E - EVALUATING ACTIONS REQUIRED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	N/A	
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	N/A	
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	A11	C/Mar 1989	SER Mar 16, 1989
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	N/A	
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	A11	C/Oct 1982	NUREG-0744, Rev. 1 T.S. Amendment #15 June 12, 1986
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWR	C/June 1978	RG&E Ltr June 26, 1978 NUREG-0577, Rev. 1 NUREG-0944
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/72 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18 (No requirements)	A11	I	1. GL 88-20, Supp 1 Response (IPE) inc commit to perform internal flooding anal 2. NUREG-0737 Item I.C.5 completed per Procedure A-1404, Jan 13, 1981
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	A11	I	1. Inspection 87-03 2. RG&E response, Mar 6, 1987 3. Insp Rpt, June 11, 1987 4. EEQ Closeout Insp 89-24 5. Insp Rpt not issued



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<u>USI, A NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOR Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	E	GL 88-11/Implement. Ltr from RG&E Jan 23, 1989 Estimate Conformance Date of May 1991
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All Ols After 01/79.	N/A	
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C/June 1985	Phase 1 SER Jan 18, 1984 Phase II Not Req'd Per GL 85-11 June 26, 1985
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SRP 6.2.1.1.C	BWR	N/A	
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	I	Integrated Into USI A-46 Resolution (SQUG)
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	N/A	

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.82 (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	A11	C/Dec 1985	GL 85-22 Excluded Operating Reactors from Requirements of Reg Guide 1.82, Rev. 1 RG&E Apr 17, 1989 Response to 10CFR50.63 Stated That Mods/Procedure Chngs to be Completed in 2 Years After Accept by Director, Office of Nuclear Regulation
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	A11	I	In Accordance with GL 88-20, Supp 1, USI A-45 to be Assessed as Part of PRA/IPE
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	A11	I	RG&E Committed to SQUG Methodology per Ltr of May 26, 1987
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	A11	I	GINNA not Explicitly Referred in GL 89-19 as Requiring Changes, GL is Being Reviewed. Scheduled Response Date is Mar 19, 1990.
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG-1218 GL 89-19	A11	E	GINNA Has Large Dry Containment
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	A11, except PWRs with large dry containments	N/A	NRC Ltr with Safety Evaluation of Nov 17, 1986
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	C/Nov 1986	



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