Docket No. 50-244

Mr. Leon D. White, Jr.
Vice President
Electric and Steam Production
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, New York 14649

NOV 16 1979

Dear Mr. White:

By letter dated April 16, 1979, we sent you the list of topics which would not be reviewed for your plant in the Systematic Evaluation Program (SEP). These topics were proposed for deletion from the SEP because they were not applicable to your plant or they were being reviewed as part of an ongoing generic issue outside the SEP. Most of the licensees of plants in the SEP provided comments on these lists. We have reviewed those comments and have conducted further internal review of our original listings. As a result, we have enclosed new listings which supersede the lists transmitted on April 16, 1979.

Enclosure 1 is the list of topics which are being reviewed as part of ongoing generic issues outside of the SEP. The SEP will only monitor the status of these topics. If the topic is completed and implemented during the course of the SEP, a topic assessment will be issued which identifies the licensing action and safety evaluation report (SER) supporting the final disposition of that topic. If an implementation position is developed but no licensing action has been taken, the issue will be considered for implementation on SEP plants on a case-by-case basis. If no implementation position has been developed by the end of the SEP, the topic assessment will merely report the status of the generic effort.

Enclosure 2 is the list of topics that have been deleted from the review of your plant because they are not applicable to your facility or site. No topic assessments will be written for these topics.

Sincerely,
"Original Signed by
Thomas V. Wambach"

SEE PREVIOUS YELLOW FOR OTHER CONCURRENCES AND DISTRIBUTION

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:
As stated
w/enclosures:
See next page

OFFICE	DOR:ORB #2	DOR:ORB #2			
SURNAME	TVWambach	DLZiemann		7912110030	
DATE	11/16/79	11/16/79			

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OCT 29 1979

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We will consider these listings final unless we hear from you within thirty days.

Sincerely,

Original signed by
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:
As stated

cc w/enclosures:
See next page

OFFICE	DOR:ORB #2	DOR:SEPB	DOR:SEPB	DOR:ORB #2	
SURNAME	TVWambach:ah	PDiBenedetto	DCrutchfield	DLZiemann	
DATE	10/26/79	10/29/79	10/29/79	10/29/79	

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ENCLOSURE 1

R. E. GINNA

SEP TOPICS NOT REQUIRING REVIEW DUE TO ONGOING GENERIC LICENSING ACTIVITIES

TOPIC III-8-A:	LOOSE PARTS MONITORING AND CORE BARREL VIBRATION MONITORING
COMMENTS:	Implementation status pending RRRC decision.
TOPIC III-8-D:	CORE SUPPORTS AND FUEL INTEGRITY
COMMENTS:	This topic is being reviewed generically by DOR under TAP A-2 for PWR's. Future A-2 under development for BWR's.
TOPIC III-9:	SUPPORT INTEGRITY
COMMENTS:	Generically reviewed under TAP A-12.
TOPIC V-1:	COMPLIANCE WITH CODES AND STANDARDS
COMMENTS:	The Engineering Branch, DOR, is performing a generic review of all plants for compliance with inspection requirements of 10 CFR 50.55a(g).
TOPIC V-3:	OVERPRESSURE PROTECTION
COMMENTS:	Listed as TASK A-26 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published date November 1978. Generic letters issued to PWR facilities in August 1976. Specific criteria has been identified to be used in the design of modifications intended to preclude exceeding the limits of 10 CFR 50, Appendix G.
TOPIC V-4:	PIPING AND SAFE END INTEGRITY
COMMENTS:	Included in the review for 10 CFR 50.55a.
TOPIC V-7:	REACTOR COOLANT PUMP OVERSPEED
COMMENTS:	Listed as TASK B-68 in NUREG-0371, dated November 1978. At this time no criteria has been developed for this task.
TOPIC V-8:	STEAM GENERATOR INTEGRITY
COMMENTS:	Listed as TASK A-3, 4, 5 of NUREG-0317, dated November 1978. The TASK ACTION PLAN mpletely envelopes the scope of this topic review.
TOPIC V-13:	WATER HAMMER
COMMENTS:	Listed as TASK A-1 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published date November 1978. Generic letter sent to PWR facilities, requirements not established for BWRs.

November 16, 1979

TOPIC VI-2-B: SUBCOMPARTMENT ANALYSIS

COMMENTS: Listed as TASK A-2 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published date November 1978. Guidance letters issued to PWR licensees October 1975. The staff is currently developing plans for expanding this activity to resolve this issue for BWR facilities (NUREG-0510). Specific requirements and scope have not been established.

TOPIC VI-6: CONTAINMENT LEAK TESTING

COMMENTS: Reviewed under TAP A-23.

TOPIC VI-7-A.2: UPPER PLENUM INJECTION

Generic review of all Westinghouse two loop plants by Analysis Branch.

TOPIC VI-7.A.4: CORE SPRAY NOZZLE EFFECTIVENESS

COMMENTS: Listed as TASK A-16 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published date November 1978. December 1976 - letter issued to licensee.

TOPIC VI-7-D: LONG TERM COOLING PASSIVE FAILURES

COMMENTS: Generically reviewed and determination as stated in NUREG-0138.

TOPIC VI-7-E: ECCS SUMP DESIGN

COMMENTS: Reviewed under TAP A-43.

TOPIC VII-1-B: TRIP UNCERTAINTY

COMMENTS: Staff evaluations, issued August 1978, state that review is being pursued independent of SEP to determine impact of instrument error and drift upon safety margins of trip setpoints, and that appropriate actions will be taken to assure that adequate safety margins are maintained.

TOPIC VII-4: FAILURES OF NON-SAFETY SYSTEMS AND SELECTED ESF's

COMMENTS: The scope of this topic is contained in TASK A-17 of NUREG-0371, November 1978. The staff is developing a method of review. Criteria has not been established for implementation.

TOPIC VII-5: INSTRUMENTS FOR MONITORING ACCIDENTS

COMMENTS: TASK A-34 of NUREG-0371, November 1978. Criteria under development.

TOPIC VII-6: FREQUENCY DECAY

COMMENTS: The scope of this topic is completely addressed in TASK A-35 (2.C-1Q) of NUREG-0371, November 1978.

TOPIC VIII-1-A: EFFECTS OF DEGRADED GRID VOLTAGE

COMMENTS: The scope of this topic is contained in TASK A-35 (C-2, 3, 4, and 5) of NUREG-0371, November 1978. Initial letters sent to licensee August 1976.

TOPIC IX-2: OVERHEAD HANDLING SYSTEMS - CRANES

COMMENTS: Reviewed under TAP A-36.

TOPIC IX-6: FIRE PROTECTION

COMMENTS: This topic is being reviewed outside the SEPB as part of the ongoing licensing review implemented as a result of the Browns Ferry incident.

TOPIC XI-1: APPENDIX I

COMMENTS: This topic is part of an ongoing licensing review for all facilities to implement the limits of the Appendix I to 10 CFR 50.

TOPIC XI-2: RADIOLOGICAL MONITORING SYSTEMS

COMMENTS: Listed in NUREG-0371, November 1978, as TASK B-67, criteria and scope of review under development.

TOPIC XIII-1: CONDUCT OF OPERATIONS

COMMENTS: Generically reviewed by DPM with the exception of "operating experience" which will be provided by I&E.

TOPIC XIII-2: SAFEGUARDS/INDUSTRIAL SECURITY

COMMENTS: Ongoing licensing review to implement the requirements of 10 CFR 73.55.

TOPIC XV-21: SPENT FUEL CASK DROP

COMMENTS: Reviewed under TAP A-36.

TOPIC XV-22: ANTICIPATED TRANSIENTS WITHOUT SCRAM

COMMENTS: TASK A-9 of NUREG-0371, November 1978, criteria for implementation under development.

TOPIC XV-23: STEAM GENERATOR MULTIPLE TUBE FAILURES

COMMENTS: This topic is being considered in conjunction with TASK A-2, 3, 4 of NUREG-0371. No criteria has been established on this topic.

TOPIC XV-24:	LOSS OF ALL AC POWER
COMMENTS:	Reviewed under TAP A-44.
TOPIC XVII:	QA FOR OPERATIONS
COMMENTS:	Reviewed by QAB for conformance with 10 CFR Appendix B.

ENCLOSURE 2

R. E. GINNA

SEP TOPICS NOT APPLICABLE TO R. E. GINNA

TOPIC NO.	TITLE	COMMENTS
II-4-E	Dam Integrity	N/A to this unit site
III-7.A	Inservice Inspection, Including Prestressed Concrete Containments With Either Grouted or Ungrouted Tendons	N/A No Current Criteria
III-10-C	Surveillance Requirements on BWR Recirculation Pumps	N/A BWR Safety Topic
IV-3	BWR Jet Pump Operating Indications	N/A BWR Safety Topic
V-2	Applicability of Code Case	N/A at this time. To be reviewed for any future modifications using references to Code Cases.
V-9	Reactor Core Isolation Cooling System	N/A BWR Safety Topic
V-12.A	Water Purity of BWR Primary Coolant	N/A BWR Safety Topic
VI-2.A	Pressure Suppression BWR Containments	N/A BWR Safety Topic
VI-2-C	Ice Condenser Containment	N/A to this unit containment design
VI-7-A-4	Core Spray Nozzle Effectiveness	N/A BWR Safety Topic
VI-7-C-3	Effect of PWR Loop Isolation Valve Closure During A LOCA on ECCS	N/A to this unit site
VI-9-A	Main Steam Line Isolation Seal System	N/A BWR Safety Topic
VI-10-B	Shared Engineer Safety Features	N/A to this unit site
VII-7	Acceptability of Swing Bus Design on BWR-4 Plants	N/A BWR Safety Topic
Group I (BWR) DBE	All	N/A BWR DBE Review
Group VI DBE	Topic XV-13 (only) Uncontrolled Rod Assembly Withdrawal at Power, Low Power Startup, Rod Drop Accident	N/A BWR DBE Review