

**28-35-135a. Definitions.** As used in these regulations, each of the following terms shall have the meaning assigned ~~assigned~~ specified in this regulation: (a) “A<sub>1</sub>” means the maximum activity of special form radioactive material permitted in a type A package.

(b) “A<sub>2</sub>” means the maximum activity of radioactive material, other than special form radioactive material, permitted in a type A package. These values either are listed in table I in K.A.R. 28-35-221b or may be derived in accordance with the procedure specified in K.A.R. 28-35-221b ~~of these regulations~~.

(c) “Absorbed dose” means the energy imparted to matter by ionizing radiation per unit mass of irradiated material at the place of interest. The units of absorbed dose are the rad and the gray (Gy).

(d) “Absorbed dose rate” means the absorbed dose per unit of time or, for linear accelerators, the dose monitor unit per unit of time.

(e) “Accelerator-produced material” means any material made radioactive by exposing it in a particle accelerator.

(f) “Accessible surface” means the surface of equipment or of an equipment part that can be easily or accidentally touched by persons without the use of a tool.

(g) “Accident” means an unintended event, including an operating error, equipment failure, and other mishap, that could result in either of the following:

(1) A dose in excess of regulatory limits on site or for the public; or

(2) consequences or potential consequences that cannot be ignored from the point of view of protection or safety, including an actual or potential substantial degradation of the level of protection or safety of the facility or the release of radioactive material in sufficient quantity to

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warrant consideration of protective actions.

(h) "Act" means the "nuclear energy development and radiation control act," K.S.A. 48-1601 et seq., and amendments thereto.

(i) "Activity" means the rate of disintegration, transformation, or decay of radioactive material. Activity is expressed in the SI unit of becquerel (Bq) or in the special unit of curie (Ci), or the multiples of either unit, ~~or disintegrations or transformations per unit of time as follows:~~

~~(1) One becquerel (Bq) equals one disintegration or transformation per second (dps or tps); and~~

~~(2) one curie (Ci) equals 3.7E+10 disintegrations or transformations per second (dps or tps). One curie also equals 3.7E+10 becquerels (Bq).~~

(j) "Added filter" means the filter added to the inherent filtration.

(k) "Address of use" means the building or buildings that are identified on the license and each location where radioactive material could be produced, prepared, received, used, or stored.

(l) "Adult" means an individual who is 18 or more years of age.

(m) ~~(1)~~ "Agreement state" means any state with which the ~~United States~~ nuclear regulatory commission (NRC) enters, or has entered, into an effective agreement pursuant to 42 U.S.C. §2021, ~~as in effect on January 4, 1995~~ subsection 274b of the atomic energy act of 1954, 68 Stat. 919, as amended.

(2) "Non-agreement state" means any other state.

(n) "Airborne radioactive area" means the following:

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(1) Any room, enclosure, or operating area in which airborne radioactive material exists in concentrations in excess of the derived air concentrations (DAC) specified in “appendices to part 4: standards for protection against radiation,” effective April 1994, published by the department and hereby adopted by reference; or

(2) any room, enclosure, or operating area in which airborne radioactive material exists in concentrations such that an individual present in the area without respiratory protective equipment could exceed, during the hours an individual is present in a week, an intake of 0.6 percent of the ALI or 12 DAC-hours.

(o) “Airborne radioactive material” means any radioactive material dispersed in the air in the form of dust, fumes, mists, vapors, or gases.

(p) “Air kerma (~~K~~)” means the kinetic energy released in air by ionizing radiation. Kerma is determined by dividing  $dE$  by  $dM$ , where  $dE$  is the sum of the initial kinetic energies of all the charged ionizing particles liberated by uncharged ionizing particles in air of mass  $dM$ . The SI unit of air kerma is joule per kilogram, and the special name for the unit of kerma is the gray (Gy).

(q) “Alert” means a period during which one of the following could lead to a release of radioactive material that is not expected to require a response by off-site response organizations to protect persons off-site:

- (1) Conditions have arisen that could cause an event.
- (2) An event is in progress.
- (3) An event has occurred.

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(r) "Aluminum equivalent" means the thickness of type 1100 aluminum alloy that affords the same attenuation, under specified conditions, as that of the material in question. The nominal chemical composition of type 1100 aluminum alloy is a minimum of 99.00 percent aluminum and 0.12 percent copper.

(s) "Amendment" means any change to a license or registration issued under these regulations.

(t) "Analytical X-ray system" means a group of local and remote components utilizing X-rays to determine the elemental composition or to examine the microstructure of materials.

(1) Local components shall include those components that are struck by X-rays, including radiation source housings, port and shutter assemblies, collimators, sample holders, cameras, goniometers, detectors, and shielding.

(2) Remote components may include power supplies, transformers, amplifiers, readout devices, and control panels.

(u) "Annual limit on intake (ALI)" ~~means~~ and "ALI" mean the derived limit for the amount of radioactive material taken into the body of an adult worker by inhalation or ingestion in a year. ALI is the smaller value of intake of a given radionuclide in a year by the reference man that would result in a committed effective dose equivalent of 5 rem (0.05 Sv) or a committed dose equivalent of 50 rem (0.5 Sv) to any individual organ or tissue. ALI values for intake by ingestion and by inhalation of selected radionuclides are specified in appendix B, table I, published in "appendices to part 4: standards for protection against radiation," which is adopted by reference in this regulation.

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(v) "Annual refresher safety training" means a review conducted or provided by the licensee or registrant for its employees on radiation safety aspects of industrial radiography. The review shall include, at a minimum, any results of internal inspections, new procedures or equipment, new or revised regulations, and accidents or errors that have been observed. The review shall also provide opportunities for employees to ask safety questions.

(w) "ANSI" means the American national standards institute.

(x) "Applicator" means a structure that determines the extent of the treatment field at a given distance from the virtual source.

(y) "Area of use" means a portion of a physical structure that has been set aside for the purpose of producing, preparing, receiving, using, or storing radioactive material.

(z) "As low as is reasonably achievable (~~ALARA~~)," and "ALARA," when used to describe exposures to radiation workers, ~~means~~ mean that every reasonable effort has been made to maintain exposures to radiation workers as far below the dose limits specified in these regulations as is practical, consistent with the purpose for which the licensed or registered activity is undertaken, taking the following into account:

- (1) The state of technology;
- (2) the economics of improvements in relation to the state of technology;
- (3) the economics of improvements in relation to benefits to public health and safety and to other societal and socioeconomic considerations; and
- (4) the economics of improvements in relation to the utilization of nuclear energy and licensed or registered sources of radiation in the public interest.

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(aa) "Assembler" means any person engaged in the business of assembling, replacing, or installing one or more components into an X-ray system or subsystem. The term shall include the owner of an X-ray system or any employee or agent of the owner who assembles components into an X-ray system that is subsequently used to provide professional or commercial services.

(bb) "Associated equipment" means equipment that is used in conjunction with a radiographic exposure device that makes radiographic exposures and that drives, guides, or comes in contact with the source.

(cc) "Attenuation block" means a block or stack, with dimensions of 20 cm by 20 cm by 3.8 cm, made of type 1100 aluminum alloy or other materials having equivalent attenuation.

(dd) "Authorized user" means an individual who is identified as an authorized user on a license issued by the department for the use of radioactive material or an individual who is designated by a registered facility as a user of X-ray machines or accelerators. This term shall not apply to part 6 of these regulations.

(ee) "Automatic exposure control" means a device that automatically controls one or more technique factors in order to obtain a required quantity of radiation, at one or more preselected locations. (Authorized by K.S.A. 48-1607; implementing K.S.A. 2016 Supp. 48-1603 and K.S.A. 48-1607; effective Dec. 30, 2005; amended July 27, 2007; amended P-\_\_\_\_\_.)

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**28-35-135c. Definitions.** As used in these regulations, each of the following terms shall have the meaning ~~assigned~~ specified in this regulation: (a) “Cabinet radiography using radiation machines” means industrial radiography that is conducted in an enclosed, interlocked cabinet that prevents the radiation machine from operating unless all openings are securely closed and that is sufficiently shielded so that every location on the cabinet’s exterior meets the conditions for an unrestricted area as specified in K.A.R. 28-35-214a.

(b) “Cabinet X-ray system” means an X-ray system with the X-ray tube installed in an enclosure, called a “cabinet,” that is independent from existing architectural structures except the floor on which the cabinet could be placed. The cabinet is intended for the following purposes:

- (1) To contain at least that portion of a material being irradiated;
- (2) to provide radiation attenuation; and
- (3) to exclude personnel from the interior of the cabinet during the generation of X-rays.

~~Cabinet X-ray systems may~~ This term shall include all X-ray systems designed primarily for the inspection of carry-on baggage at airline, railroad, and bus terminals, and in similar facilities. An X-ray tube that is used within a shielded part of a building, or X-ray equipment that may temporarily or occasionally incorporate portable shielding, shall not be considered a cabinet X-ray system.

(c) “Calendar quarter” means at least 12 but not more than 14 consecutive weeks. The first calendar quarter of each year shall begin in January. Subsequent calendar quarters shall be arranged so that no day is included in more than one calendar quarter and no day in any one year is omitted from inclusion within a calendar quarter. A licensee or registrant shall not change the method of determining and observing calendar quarters for purposes of these regulations except

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at the beginning of a calendar year.

(d) "Calibration" means the determination of either of the following:

(1) The response or reading of an instrument relative to a series of known radiation values over the range of the instrument; or

(2) the strength of a source of radiation relative to a standard.

(e) "Camera" means a radiographic exposure device.

(f) "Central axis of the beam" means a line passing through the virtual source and the center of the plane figure formed by the edge of the first beam-limiting device.

(g) "Cephalometric device" means a device intended for the radiographic visualization and measurement of the dimensions of the human head.

(h) "Certifiable cabinet X-ray system" means an existing, uncertified X-ray system that has been modified to meet the certification requirements specified in 21 ~~CFR~~ C.F.R. 1020.40, as in effect on April 30, 1984.

(i) "Certified cabinet X-ray system" means a cabinet X-ray system that has been certified as manufactured and assembled as specified in 21 ~~CFR~~ C.F.R. 1020.40, as in effect on April 30, 1984.

(j) "Certified components" means the components of X-ray systems that are subject to regulations promulgated under public law 90-602, the radiation control for health and safety act of 1968 ~~and amendments thereto~~ as amended.

(k) "Certified system" means any X-ray system that has one or more certified components.

(l) "Certifying entity" means an independent certifying organization or state regulatory

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program meeting the requirements in K.A.R. ~~28-35-292~~ 28-35-293.

(m) "Changeable filter" means any filter, exclusive of inherent filtration, that can be removed from the useful beam through any electronic, mechanical, or physical process.

(n) "Chelating agent" means amine polycarboxylic acids, hydroxycarboxylic acids, gluconic acids, and polycarboxylic acids.

(o) "Class" means a classification scheme for inhaled material according to its rate of clearance from the pulmonary region of the lung. For the purposes of these regulations, "lung class" and "inhalation class" shall be considered equivalent terms. Materials are classified as D, W, or Y, which applies to the following range of clearance half-times:

- (1) For class D, fewer than 10 days;
- (2) for class W, from 10 through 100 days; and
- (3) for class Y, more than 100 days.

(p) "Coefficient of variation" ~~or~~ and "C" ~~means~~ mean the ratio of the standard deviation to the mean value of a population of observations. This ratio is estimated using the following equation:

$$C = \frac{s}{\bar{x}} = \frac{1}{\bar{x}} \left( \frac{\sum_{i=1}^n (x_i - \bar{x})^2}{n - 1} \right)^{\frac{1}{2}}$$

where

s = Estimated standard deviation of the population-

$\bar{x}$  = Mean value of observations in sample-

$x_i$  = ith observation in sample

(q) "Collective dose" means the sum of the individual doses received in a given period of

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time by a specified population from exposure to a specified source of radiation.

(r) “Collimator” means a radiation shield that is placed at the end of a guide tube or directly onto a radiographic exposure device to restrict the size of the radiation beam when the sealed source is cranked into position to make a radiographic exposure.

(s) “Committed dose equivalent ( $H_{T,50}$ )” ~~means~~ and “ $H_{T,50}$ ” mean the dose equivalent to organs or tissues of reference (T) that will be received from an intake of radioactive material by an individual during the 50-year period following the intake.

(t) “Committed effective dose equivalent ( $H_{E,50}$ )” ~~means~~ and “ $H_{E,50}$ ” mean the sum of the products of the weighting factors applicable to each of the body organs or tissues that are irradiated and the committed dose equivalent to each of these organs or tissues ( $H_{E,50} = \sum w_T H_{T,50}$ ).

(u) “Computed tomography” means the production of a tomogram by the acquisition and computer processing of X-ray transmission data.

(v) “Consortium” means an association of medical use licensees and a positron emission tomography (PET) radionuclide production facility in the same geographical area that jointly own or share the operation and maintenance cost of the PET radionuclide production facility that produces PET radionuclides for use in producing radioactive drugs within the consortium for noncommercial distributions among its associated members for medical use.

(w) “Contact therapy” means therapy in which the X-ray tube port is put in contact with, or within five centimeters of, the surface being treated.

(~~w~~) (x) “Contact therapy system” means a therapeutic radiation machine with a short target-to-skin distance (TSD), usually less than five centimeters.

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~~(x)~~ (y) "Control cable" means the cable that is connected to the source assembly and used to drive the source to and from the exposure location.

~~(y)~~ (z) "Control drive mechanism" means a device that enables the source assembly to be moved into and out of the exposure device.

~~(z)~~ (aa) "Controlled area" means an area outside of a restricted area but inside the site boundary, access to which can be limited by the licensee or registrant for any reason.

~~(aa)~~ (bb) "Control panel" means that part of the X-ray system where the switches, knobs, push buttons, and other hardware necessary for manually setting the technique factors are mounted.

~~(bb)~~ (cc) "Control tube" means a protective sheath for guiding the control cable. The control tube connects the control drive mechanism to the radiographic exposure device.

~~(cc)~~ (dd) "Cooling curve" means the graphical relationship between the heat units stored and the cooling time.

~~(dd)~~ (ee) "Curie" means a unit of activity. One curie (Ci) is the quantity of radioactive material that decays at the rate of  $3.7 \times 10^{10}$  transformations per second (tps). Commonly used submultiples of the curie are the millicurie and the microcurie. One millicurie ( $mCi$ ) = 0.001 curie =  $3.7 \times 10^7$  tps. One microcurie ( $\mu Ci$ ) = 0.000001 curie =  $3.7 \times 10^4$  tps. (Authorized by K.S.A. 48-1607; implementing K.S.A. 2016 Supp. 48-1603 and K.S.A. 48-1607; effective Dec. 30, 2005; amended P-\_\_\_\_\_.)

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**28-35-135i. Definitions.** As used in these regulations, each of the following terms shall have the meaning ~~assigned~~ specified in this regulation: (a) “Image intensifier” means a device that instantaneously converts, by means of photoemissive surfaces and electronic circuitry, an X-ray pattern into a light pattern of greater intensity than would have been provided by the original X-ray pattern.

(b) “Image receptor” means any device, including a fluorescent screen and radiographic film, that transforms incident X-ray photons into a visible image or into another form that can be made into a visible image by further transformations.

(c) “Image receptor support,” for mammographic systems, means that part of the system designed to support the image receptor in a horizontal plane during a mammographic examination.

(d) “Immediate” means within not more than 15 minutes or as otherwise defined in a license condition.

(e) “Incident” means an individual event or series of related events that caused or threatened to cause any violation of these regulations or license conditions. For the purposes of part 13, “incident” ~~means~~ shall mean any unintended event involving radioactive material for which the public dose is a fraction of regulatory limits and safety provisions are sufficient, but further degradation of safety systems could lead to an accident.

(f) “Independent certifying organization” means an independent organization that meets all of the criteria specified in K.A.R. ~~28-35-292~~ 28-35-293.

(g) “Indian tribe” and “tribe” mean any Indian tribe, band, nation, or other organized

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group or community of Indians recognized as eligible for the services provided to Indians by the secretary of the United States department of the interior because of their status as Indians.

(h) "Indian tribal official" and "tribal official" mean the highest-ranking individual who represents tribal leadership, including the chief, president, and tribal council leader.

(i) "Individual" means any human being.

~~(h)~~ (j) "Individual monitoring" means the assessment of either of the following:

(1) A dose equivalent by the use of individual-monitoring devices or by the use of survey data; or

(2) a committed effective dose equivalent determined by bioassay or by computation of the number of DAC-hours to which an individual is exposed.

~~(i)~~ (k) "Individual-monitoring device" means any device designed to be worn by a single individual for the assessment of dose equivalent. "Individual-monitoring device" shall include any film badge, thermoluminescent dosimeter (TLD), optically stimulated dosimeter, pocket ionization chamber, and personal air-sampling device. For purposes of these regulations, "personnel dosimeter" and "dosimeter" shall be considered terms equivalent to "individual-monitoring device."

~~(j)~~ (l) "Industrial radiography" means the examination of the structure of materials by nondestructive methods utilizing sources of radiation.

~~(k)~~ (m) "Inherent filtration" means the filtration permanently mounted in the useful beam, including the window of the X-ray tube and any permanent tube or source enclosure.

~~(l)~~ (n) "Injection tool" means a device used for controlled subsurface injection of

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radioactive tracer material.

~~(m)~~ (o) “Inspection” means an official examination or observation that may include tests, surveys, and monitoring to determine compliance with federal rules, state regulations, orders, requirements, and license and registration conditions.

~~(n)~~ (p) “Installation” means the location where one or more sources of radiation are used, operated, or stored.

~~(o)~~ (q) “Interlock” means a device for precluding access by an individual to an area of radiation hazard without warning, either by preventing admission or by automatically removing the hazards.

~~(p)~~ (r) “Internal dose” means that portion of the dose equivalent received from radioactive material taken into the body.

~~(q)~~ (s) “Interruption of irradiation” means the stopping of irradiation with the possibility of continuing irradiation without the resetting of operating conditions at the control panel.

~~(r)~~ (t) “Ionizing radiation” means radiation capable of producing an ionization event, including gamma rays and X-rays, alpha and beta particles, high-speed electrons, neutrons, and other nuclear particles.

~~(s)~~ (u) “Irradiation” means the exposure of matter to ionizing radiation.

~~(t)~~ (v) “Irradiator” means a facility that uses radioactive sealed sources for the irradiation of objects or materials and in which radiation dose rates exceeding five grays (500 rads) per hour exist at one meter from the sealed radioactive sources in air or water, as applicable for the irradiator type. This term shall not include any irradiator in which both the sealed source and the

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area subject to irradiation are contained within a device and are not accessible to personnel.

(u) (w) "Irradiator operator" means an individual who has successfully completed the required training and testing and is authorized by the terms of the license to operate an irradiator without a supervisor present.

(v) (x) "Irretrievable well-logging source" means any sealed source containing licensed material that is pulled off or not connected to the wireline that suspends the source in the well and for which all reasonable effort at recovery has been expended.

(w) (y) "Isocenter" means a fixed point in space that is located at the center of the smallest sphere through which the central axis of the beams passes under all conditions. (Authorized by K.S.A. 48-1607; implementing K.S.A. 2016 Supp. 48-1603 and K.S.A. 48-1607; effective Dec. 30, 2005; amended P-\_\_\_\_\_.)

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**28-35-135s. Definitions.** As used in these regulations, each of the following terms shall have the meaning assigned specified in this regulation: (a) "Sanitary sewerage" means a system of public sewers to carry off waste water and refuse. This term shall exclude sewage treatment facilities, septic tanks, and leach fields owned or operated by the licensee or registrant.

(b) "Scattered radiation" means radiation that, during its passage through matter, is deviated in direction.

(c) "Sealed source" means any radioactive material that is permanently encased in a capsule designed to prevent the leakage or escape of the radioactive material.

(d) "Secondary dose-monitoring system" means a system that terminates irradiation if the primary system fails.

(e) "Secondary protective barrier" means a barrier sufficient to attenuate stray radiation to the required degree.

(f) "Secretary" means ~~the~~ secretary of the department of health and environment.

(g) "Seismic area" means any area where the probability of a horizontal acceleration in rock of more than 0.3 times the acceleration of gravity in 250 years is greater than 10 percent, as designated by the U.S. geological survey.

(h) "Shallow dose equivalent ( $H_s$ )," and " $H_{s,2}$ " which ~~applies~~ apply to the external exposure of the skin or an extremity, ~~means~~ mean the dose equivalent at a tissue depth of 0.007 centimeter ( $7 \text{ mg/cm}^2$ ) averaged over an area of one square centimeter.

(i) "Sheltering" means using a structure for radiation protection from an airborne plume containing radioactive material.

(j) "Shielded position" means the location within the radiographic exposure device or

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storage container that, by the manufacturer's design, is the proper location for storage of the sealed source.

(k) "Shielded-room radiography using radiation machines" means industrial radiography using radiation machines that meets the following conditions:

(1) Is conducted in an enclosed room, the interior of which is not occupied during radiographic operations;

(2) is shielded so that every location on the exterior meets the conditions specified in K.A.R. 28-35-214a; and

(3) is accessible only through openings that are interlocked so that the radiation machine will not operate unless all openings are securely closed.

(l) "SI" means the abbreviation for the international system of units.

(m) "Shutter" means a device attached to an X-ray tube housing assembly that can totally intercept the useful beam and that has a lead equivalency not less than that of the tube housing assembly.

(n) "Sievert" means the SI unit of any of the quantities expressed as a dose equivalent. The dose equivalent in sieverts is equal to the absorbed dose in grays multiplied by the quality factor (1 Sv = 100 rem).

(o) "Site area emergency" means an event that could occur, is in progress, or has occurred, that could lead to a significant release of radioactive material, and that could require a response by off-site response organizations to protect persons off-site.

(p) "Site boundary" means that line beyond which the land or property is not owned,

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leased, or otherwise controlled by the licensee or registrant.

(q) "Source" means the focal spot of the X-ray tube.

(r) "Source assembly" means an assembly that consists of the sealed source and a connector that attaches the source to the control cable.

(s) "Source changer" means a device designed and used for replacement of sealed sources in radiographic exposure devices, including those devices also used for transporting and storing sealed sources.

(t) "Source holder" means a housing or assembly into which a radioactive source is placed for the purpose of facilitating the handling and use of the source in well-logging operations.

(u) "Source-image receptor distance" and "SID" mean the distance from the source to the center of the input surface of the image receptor.

(v) "Source material" means the following:

(1) Uranium or thorium, or any combination of these, in any physical or chemical form;

or

(2) ores that contain, by weight, 0.05 percent or more of uranium, thorium, or any combination of these.

The term "source material" shall not include special nuclear material.

(w) "Source material milling" means any activity that results in the production of by-product material.

(x) "Source of radiation" means any material, device, or equipment that emits or is

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capable of producing radiation.

(y) “Source-to-skin distance” and “SSD” mean the distance between the source and the patient’s skin.

(z) “Special form” means any licensed material that meets either of the following conditions:

(1)(A) Is in solid form;

(B) has at least one dimension measuring at least five millimeters;

(C) does not melt, sublime, or ignite in air at a temperature of 1,000° F;

(D) does not shatter or crumble if subjected to the percussion test described in K.A.R. 28-35-144; and

(E) is not dissolved or converted into dispensable form to the extent of more than 0.005 percent by weight by immersion for one week in water at 68° F or in air at 86° F; or

(2)(A) Is in any physical form securely contained in a capsule;

(B) has at least one dimension measuring at least five millimeters;

(C) will retain its contents if subjected to the tests described in K.A.R. 28-35-144; and

(D) is constructed of materials that do not melt, sublime, or ignite in air at 1,475° F and do not dissolve or convert into dispensable form to the extent of more than 0.005 percent by weight by immersion for one week in water at 68° F or in air at 86° F.

(aa) “Special nuclear material” means either of the following:

(1) Plutonium, uranium-223, uranium enriched in the isotope 233 or in the isotope 235, and any other material that the department declares by order to be special nuclear material

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after the ~~U.S.~~ nuclear regulatory commission (NRC), pursuant to the provisions of section 51 of the atomic energy act of 1954, has determined the material to be special nuclear material, except for source material; or

(2) any material artificially enriched as specified in paragraph (aa)(1), except for source material.

(bb) "Special nuclear material in quantities not sufficient to form a critical mass" means any of the following:

(1) Uranium enriched in the isotope U-235, in quantities not exceeding 350 grams of contained U-235;

(2) uranium enriched in the isotope uranium-233, in quantities not exceeding 200 grams of contained U-233;

(3) plutonium not exceeding 200 grams; or

(4) any combination of these special nuclear materials in accordance with the following formula:

$$\frac{\text{grams of contained U-235}}{350} + \frac{\text{grams of contained U-233}}{200} + \frac{\text{gram of Pu}}{200} \leq 1$$

The sum of the ratios for all of the kinds of special nuclear material in combination shall not exceed one.

(cc) "Spot check" means a procedure that is performed to ensure that a previous

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calibration continues to be valid.

(dd) "Spot film" means a radiograph that is made during a fluoroscopic examination or radiation therapy treatment to permanently record conditions that exist during the procedure.

(ee) "Spot-film device" means a device intended either to transport and position a radiographic image receptor between the radiation source and image receptor or to position a radiographic image receptor between the radiation source and image receptor. This term shall include a device intended to hold a cassette over the input end of an image intensifier for the purpose of making a radiograph.

(ff) "Stationary beam therapy" means radiation therapy without relative displacement of the useful beam and the patient during irradiation.

(gg) "Stationary X-ray equipment" means X-ray equipment that is installed in a fixed location.

(hh) "Stereotactic radiosurgery" means the use of external radiation in conjunction with a stereotactic guidance device to very precisely deliver a therapeutic dose to a tissue volume.

(ii) "Stochastic effect" means a health effect that occurs randomly and for which the probability of the occurrence of the effect, rather than the severity of the effect, is assumed to be a linear function of dose without threshold. For purposes of these regulations, "probabilistic effect" shall be considered an equivalent term.

(jj) "Storage area" means any location, facility, or vehicle that is used to store, transport, or secure a radiographic exposure device, radiation machine, storage container, or sealed source when not in use. Each storage area shall be locked or have physical barriers to prevent accidental exposure,

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tampering, or unauthorized removal of the device, machine, sealed source, or container.

(kk) "Storage container" means a device in which radioactive materials are transported or stored.

(ll) "Stray radiation" means the sum of leakage radiation and scattered radiation.

(mm) "Structured educational program" means an educational program designed to impart particular knowledge and practical education through interrelated studies and supervised training.

(nn) "S-tube" means a tube through which the radioactive source travels when inside a radiographic exposure device.

(oo) "Subsurface studies" means the evaluation of parameters below the surface of the earth.

(pp) "Subsurface tracer study" means the release of a substance tagged with radioactive material for the purpose of tracing the movement or position of the tagged substance in the well bore or adjacent formation.

(qq) "Survey" means an evaluation of a radiation hazard resulting from the production, use, transfer, release, disposal, or presence of sources of radiation. This term shall include a physical survey of the location of materials or equipment, or both, and either the measurements of levels of radiation ~~contamination~~ or the concentrations or quantities of radioactive materials present.

(Authorized by K.S.A. 48-1607; implementing K.S.A. 2016 Supp. 48-1603 and K.S.A. 48-1607; effective Dec. 30, 2005; amended P-\_\_\_\_\_.)

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**28-35-135u. Definitions.** As used in these regulations, each of the following terms shall have the meaning ~~assigned~~ specified in this regulation: (a) “Underwater irradiator” means an irradiator in which the sources always remain shielded underwater and humans do not have access to the sealed sources or the space that is subject to irradiation without entering the pool.

(b) “Underwater radiography” means industrial radiography performed when the radiographic exposure device or the related equipment is beneath the surface of the water.

(c) “Unit dose” means a dosage prepared for medical use for administration to a patient or human research subject as a single dosage, without any further manipulation of the dosage after the dosage is initially prepared.

(d) “Unrefined and unprocessed ore” means ore in its natural form before any processing, including grinding, roasting, beneficiating, ~~or~~ and refining. “Processing” shall not include sieving or the encapsulation of ore or preparation of samples for laboratory analysis.

(e) “Unrestricted area” means an area to which access is neither limited nor controlled by the licensee or registrant. For purposes of these regulations, “uncontrolled area” shall be considered an equivalent term.

(f) “Useful beam” means the part of the radiation that passes through a window, aperture, cone, or other collimating device. (Authorized by K.S.A. 48-1607; implementing K.S.A. 2016 Supp. 48-1603 and K.S.A. 48-1607; effective Dec. 30, 2005; amended P-\_\_\_\_\_.)

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**28-35-140. Exemptions.** (a) *General provision.* ~~The secretary, upon application for an exemption or upon the secretary's own initiative, may grant exemptions or exceptions from the requirements of these regulations, if it is determined that the exemption will not result in an undue hazard to public health and safety, or to property.~~

(b) Carriers. ~~Common and contract carriers, freight forwarders, and warehousemen, who are subject to the rules and regulations of the U.S. department of transportation or the U.S. postal service (39 CFR Parts 14 and 15), shall be exempt from these regulations to the extent that they transport or store sources of radiation in the regular course of their carriage for another. Private carriers who are subject to the rules and regulations of the U.S. department of transportation shall be exempt from these regulations to the extent that they transport sources of radiation. Common, contract, and private carriers who are not subject to the rules and regulations of the U.S. department of transportation or the U.S. postal service shall be subject to applicable sections of these regulations. Each common carrier, each contract carrier, each freight forwarder, and each U.S. postal service carrier that only transports or stores radioactive material in the regular course of carriage or storage shall be exempt from parts 3, 4, 6, 7, 10, 11, and 12 of these regulations and from K.A.R. 28-35-700.~~

(e) (b) U.S. department of energy contractors and U.S. nuclear regulatory commission contractors. ~~Any~~ Each U.S. department of energy contractor or subcontractor and ~~any~~ each U.S. nuclear regulatory commission contractor or subcontractor operating ~~within this state~~ in Kansas shall be exempt from these regulations to the extent that the contractor or subcontractor, under the contract, receives, possesses, uses, transfers, or acquires sources of radiation; and if the contractor or subcontractor is included in one of the following categories:

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(1) Prime contractors performing work for the U.S. department of energy at sites owned or controlled by the U.S. government-owned or controlled sites government, including the transportation of sources of radiation to or from ~~such~~ these sites and the performance of contract services during temporary interruptions of ~~such~~ transportation;

(2) prime contractors of the U.S. department of energy performing research in, or development, manufacture, storage, testing, or transportation of, atomic weapons or components of atomic weapons;

(3) prime contractors of the U.S. department of energy using or operating nuclear reactors or other nuclear devices in a ~~United States~~ U.S. government-owned vehicle or vessel; and

(4) any other prime contractor or subcontractor of the U.S. department of energy or the U.S. nuclear regulatory commission ~~when~~ if the secretary determines that, under the terms of the contract or subcontract, there is adequate assurance that the work can be accomplished without undue risk to the public health and safety. (Authorized by and implementing K.S.A. 1984 ~~Supp.~~ 48-1607; effective Jan. 1, 1970; amended, T-85-43, Dec. 19, 1984; amended May 1, 1985; amended P-\_\_\_\_\_.)

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**28-35-177a. General licenses; source material.** ~~(a)(1) Each of the following persons shall be deemed to have been issued a general license authorizing the acquisition, possession, use, and transfer of not more than 15 pounds (6.8 kg) of source material at any one time or the receipt of a total of 150 pounds (68.2 kg) of source material in any calendar year if the source material is used for research, development, education, commercial, or operational purposes:~~

~~(A) Any commercial or industrial firm;~~

~~(B) any research, educational, or medical institution; and~~

~~(C) any state or local governmental agency.~~

~~(2) Each person who acquires, possesses, uses, or transfers source material pursuant to the general license specified in subsection (a) shall be exempt from parts 4 and 10 of these regulations to the extent that the acquisition, possession, use, or transfer is within the terms of the general license. This exemption shall not apply to any person who is also in possession of source material under a specific license issued pursuant to these regulations.~~

~~(3) Each person who receives, possesses, uses, or transfers source material pursuant to the general license specified in subsection (a) shall be prohibited from administering source material or the radiation, either externally or internally, to human beings except as may be authorized in a specific license.~~

~~(b) Each person receiving title to source material shall be deemed to have been issued a general license without regard to quantity. This general license shall not authorize any person to receive, possess, use, or transfer source material.~~

~~(c)(1) Each person who meets the requirements of paragraphs (2), (3), and (4) of this subsection shall be deemed to have been issued a general license to acquire, possess, use, or~~

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~~transfer depleted uranium contained in industrial products or devices for the purpose of providing a concentrated mass in a small volume of the product or device.~~

~~(2)(A) Each person who acquires, possesses, or uses depleted uranium pursuant to the general license issued in this subsection shall file a form specified by the department. The form shall be filed with the department within 30 days of the date on which the depleted uranium is received or acquired. Each person filing a form shall provide all the information requested by the form.~~

~~(B) If any change in circumstances renders any information provided on the form inaccurate, the department shall be provided with a written notice of the change within 30 days of the date of the change.~~

~~(3) A person who acquires, possesses, or uses depleted uranium pursuant to the general license specified in this subsection shall not perform any of the following:~~

~~(A) Introduce depleted uranium, in any form, into a chemical, physical, or metallurgical treatment or process, except a treatment or process for the repair or restoration of any plating or other covering of the depleted uranium;~~

~~(B) abandon depleted uranium; or~~

~~(C) export depleted uranium, except in accordance with a license issued by the U.S. nuclear regulatory commission.~~

~~(4)(A) Each person possessing depleted uranium pursuant to the general license specified in this subsection shall transfer or dispose of the depleted uranium only by transfer in accordance with K.A.R. 28-35-190a.~~

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~~(B) When depleted uranium is transferred to any person in this state, the transferor shall provide a copy of this regulation and the required form to the transferee.~~

~~(C) When depleted uranium is transferred to any person outside this state, the transferor shall furnish the transferee with a copy of this regulation, the required form, and a written notice that possession or use of the depleted uranium is regulated by the U.S. nuclear regulatory commission or the state in which the person is located, under requirements substantially the same as those in this regulation.~~

~~(D) Each person who transfers depleted uranium pursuant to this subsection shall give written notice to the department of the name and address of the person to whom the depleted uranium was transferred. The notice shall be filed within 30 days of the date of transfer.~~

~~(5) The general license specified in this subsection shall apply only to industrial products or devices that have been manufactured or initially transferred in accordance with a specific license that authorizes the manufacture of the products or devices for distribution to persons generally licensed by the NRC or an agreement state.~~

~~(d) Each person who acquires, possesses, uses, or transfers depleted uranium pursuant to subsection (c) shall be exempt from parts 4 and 10 of these regulations with respect to the depleted uranium acquired, possessed, used, or transferred by that person. A general license is hereby issued authorizing commercial and industrial firms, research, educational, and medical institutions, and federal, state, and local government agencies to receive, possess, use, and transfer uranium and thorium, in their natural isotopic concentrations and in the form of depleted uranium, for research, development, educational, commercial, or operational purposes in any of~~

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the following forms and quantities:

(1) Uranium and thorium in the following quantities and forms:

(A) Not more than 1.5 kg (3.3 lb) of uranium and thorium in dispersible forms, including gases, liquids, and powders, at any one time. All material processed by the general licensee that alters the chemical or physical form of the material containing source material shall be accounted for as a dispersible form. A person authorized to possess, use, and transfer source material under this paragraph shall not receive more than a total of 7 kg (15.4 lb) of uranium and thorium in any one calendar year; and

(B) not more than 7 kg (15.4 lb) of uranium and thorium at any one time. A person authorized to possess, use, and transfer source material under this paragraph shall not receive more than 70 kg (154 lb) of uranium and thorium in any one calendar year. A person shall not alter the chemical or physical form of the source material possessed under this paragraph unless the source material is accounted for under the limits of paragraph (a)(1);

(2) not more than 7 kg (15.4 lb) of uranium, removed during the treatment of drinking water, at any one time. A person shall not remove more than 70 kg (154 lb) of uranium from drinking water during a calendar year under this paragraph; or

(3) not more than 7 kg (15.4 lb) of uranium and thorium at laboratories for the purpose of determining the concentration of uranium and thorium contained within the material being analyzed at any one time. A person authorized to possess, use, and transfer source material under this paragraph shall not receive more than 70 kg (154 lb) of source material in any one

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calendar year.

(b)(1) Each person who receives, possesses, uses, or transfers source material in accordance with the general license in subsection (a) shall be prohibited from the following:

(A) Administering source material, or the radiation from the source material, either externally or internally, to human beings except as authorized by a specific license issued by the department;

(B) abandoning the source material. Source material may be disposed of as follows:

(i) A cumulative total of 0.5 kg (1.1 lb) of source material in a solid, nondispersible form may be transferred each calendar year by a person authorized to receive, possess, use, and transfer source material under the general license to persons receiving the material for permanent disposal. The recipient of source material transferred under this paragraph shall be exempt from the requirements to obtain a license under part 3 of these regulations to the extent that the source material is permanently disposed of. This exemption shall not apply to any person who is in possession of source material under a specific license issued by the department; or

(ii) source material may be disposed in accordance with K.A.R. 28-35-190a; and

(C) exporting the source material to another country except in accordance with a license issued by the nuclear regulatory commission (NRC).

(2) Each person specified in paragraph (b)(1) shall respond to each written request from the department to provide information relating to the general license within 30 calendar days of the date of the request or other time specified in the request. If the person cannot provide the

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requested information within the required time, the person shall, within the same time period, request a longer period to supply the information by providing the department with a written justification for the request.

(c) Each person who receives, possesses, uses, or transfers source material in accordance with subsection (a) shall minimize contamination of the facility and the environment. When activities involving source material are permanently ceased at any site, if evidence of significant contamination is identified, the general licensee shall notify the department about the contamination and may consult with the department regarding the appropriateness of sampling and restoration activities to ensure that any contamination or residual source material remaining at the site where source material was used under this general license is not likely to result in exposures that exceed the limits in these regulations.

(d) Each person who receives, possesses, uses, or transfers source material in accordance with the general license granted in subsection (a) shall be exempt from parts 4 and 10 of these regulations to the extent that the receipt, possession, use, and transfer are within the terms of this general license, except that the person shall meet the requirements of paragraph (b)(1)(B) and subsection (c). This exemption shall not apply to any person who also holds a specific license issued by the department.

(e) No person shall initially transfer or distribute source material to persons generally licensed under paragraph (a)(1) or (2) or equivalent regulations of an agreement state, unless authorized by a specific license issued by the NRC or equivalent provisions of an agreement state. This subsection shall not apply to analytical laboratories returning any processed sample to

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the client who initially provided the sample. (Authorized by and implementing K.S.A. 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended Dec. 30, 2005; amended July 27, 2007; amended P-\_\_\_\_\_.)

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**28-35-178i. General licenses for certain units of radium-226.** (a) Subject to the limitations ~~prescribed in subsection~~ subsections (b), (c) and (d) of this regulation, a general license is hereby issued to ~~commercial and industrial firms, and to research, educational, medical and governmental institutions,~~ any person to acquire, possess, use, and transfer radium-226 in ~~units not exceeding 0.1 microcurie each.~~ contained in the following products if manufactured before the effective date of this regulation:

(1) Antiquities originally intended for use by the general public. For the purposes of this paragraph, "antiquities" shall mean products originally intended for use by the general public and distributed in the late 19th and early 20th centuries, including radium emanator jars, revigators, radium water jars, radon generators, refrigerator cards, radium bath salts, and healing pads;

(2) intact timepieces containing more than 0.037 megabecquerel (1 microcurie), nonintact timepieces, and timepiece hands and dials no longer installed in timepieces;

(3) luminous items installed in air, marine, or land vehicles;

(4) all other luminous products not listed in this subsection, if not more than 100 items are used or stored at the same location at any one time; and

(5) small radium sources containing not more than 0.037 megabecquerel (1 microcurie) of radium-226.

(b) A person shall not acquire, possess, use, or transfer radium-226 pursuant to the general license issued in subsection (a) ~~of this regulation~~ until the person has filed form RH-37 with the secretary and has received from the secretary a validated copy of the form, with a certification number assigned. Each person filing a form RH-37 shall provide all the information required by that form.

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(c) ~~Each general licensee under this regulation:~~ Each person who acquires, receives, possesses, uses, or transfers by-product material in accordance with the general license issued in subsection (a) shall meet the following requirements:

(1) ~~Shall not possess, at any one time and at any one location of storage or use, a total amount of radium-226 in excess of five microcuries~~ Notify the department of any indication of possible damage to the product that indicates a potential loss of the radioactive material. A report containing a brief description of the event and the remedial action taken shall be provided to the department within 30 days of the incident;

(2) ~~shall store the radium-226, until used, in the original shipping container or in a container providing equivalent radiation protection~~ not abandon any products containing radium-226. The product and any radioactive material from the product shall be disposed of only according to K.A.R. 28-35-165 or by transfer to a person authorized by a specific license to receive the radium-226 in the product or as otherwise approved by the department;

(3) ~~shall transfer the radioactive material only to a person who is authorized to receive it pursuant to a license issued by the secretary, the United States nuclear regulatory commission or an agreement state~~ not export any products containing radium-226 except in accordance with K.A.R. 28-35-178b; and

(4) ~~shall not transfer the radioactive material in any manner other than in the unopened, labeled shipping container as received from the shipper.~~ dispose of any products containing radium-226 at a disposal facility authorized to dispose of radioactive material in accordance with any federal or state solid or hazardous waste law, including the solid waste disposal act of 1965,

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42 U.S.C. 6901 through 6992k as amended, as authorized under 42 U.S.C. 15801 et seq., by transfer to a person authorized to receive radium-226 by a specific license issued under K.A.R. 28-35-180a or equivalent regulations of an agreement state, or as otherwise approved by the department; and

(5) respond to any written request from the department to provide information relating to the general license within 30 calendar days of the date of the request or other time specified in the request. If the general licensee cannot provide the requested information within the allotted time, that licensee shall, within that same time period, request a longer period to supply the information by submitting a letter to the department and shall provide written justification as to why the person cannot comply.

(d) Each general licensee under this regulation shall file with the secretary a written report ~~with the secretary~~ of any changes in the information filed in form RH-37. The report shall be furnished within 30 days after the effective date of the change.

(e) Each general licensee under this regulation shall be exempt from the requirements of parts 4 and 10 of these regulations with respect to the radioactive material covered by the general license.

(f) ~~The general license issued in this regulation shall not authorize the manufacture, commercial distribution or human use of radium-226.~~ The general license specified in subsection (a) shall not authorize the manufacture, assembly, disassembly, repair, or import of any products containing radium-226, except that timepieces may be disassembled and repaired.

(g) Any general licensee under this regulation who is an individual member of the public

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may submit an application to the department for a waiver from the general license fee prescribed in K.A.R. 28-35-147a. (Authorized by and implementing K.S.A. 1984 Supp. 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended P- \_\_\_\_\_.)

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**28-35-179a. Application for specific license; renewal or amendment.** (a) Any person may file a written application with the secretary for a specific license to acquire, possess, use, or transfer radioactive material, ~~and~~. Each person shall file a written application with the secretary to renew or amend any specific license. Each application for a specific license, or a renewal or an amendment of an existing license, shall be ~~made~~ submitted on the appropriate form ~~prescribed and~~ furnished by the secretary. Each person filing an application shall provide all the information requested on the application form, and any additional relevant information requested by the secretary.

(b) Each application filed with the secretary shall be signed by the applicant or licensee, or by a person authorized to act for or on behalf of the applicant or licensee.

(c) Any application may incorporate, by reference, information provided in applications, reports, or other documents previously filed with the secretary. ~~Any~~ Each reference to information previously filed with the secretary shall be clear and specific.

(d) ~~An~~ Any application for a specific license may include a request for a license authorizing activity at one or more installations or locations.

(e) Except as provided in subsections (f), (g), and (h), each application for a specific license to use radioactive material in the form of a sealed source or in a device that contains the sealed source shall include either of the following:

(1) Identification of the sealed source or device by manufacturer and model number as registered with the department, nuclear regulatory commission (NRC), or an agreement state; or

(2) sufficient information about the design, manufacture, prototype testing, quality control program, labeling, proposed uses, and leak testing to provide reasonable assurance that the radiation safety properties of the sealed source or device are adequate to protect health and minimize danger to life and property. For a device, the application shall also include sufficient

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information about installation, service and maintenance, operating and safety instructions, and potential hazards, to provide reasonable assurance that the radiation safety properties of the sealed source or device are adequate to protect health and minimize danger to life and property.

(f) For any sealed source or device manufactured before October 23, 2012 that is not registered with the department, NRC, or an agreement state and for which the applicant is unable to provide the information specified in this regulation, the application shall include the following:

(1) All available information specified in K.A.R. 28-35-181e, concerning the sealed source, and, if applicable, the device; and

(2) sufficient additional information to demonstrate reasonable assurance that the radiation safety properties of the sealed source or device are adequate to protect health and minimize danger to life and property. The information shall include a description of the sealed source or device, a description of radiation safety features, the intended use and associated operating experience, and the results of the most recent leak test.

(g) For sealed sources and devices allowed to be distributed without the registration of safety information as required in this regulation, the applicant may supply only the name of the manufacturer, model number, and radionuclide quantity.

(h) If it is not feasible to identify each sealed source and device individually, the applicant may propose constraints on the number and type of sealed sources and devices to be used and the conditions under which the sealed sources and devices will be used, instead of identifying each sealed source and device. (Authorized by and implementing K.S.A. 1984 Supp. 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended P-\_\_\_\_\_.)

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**28-35-180a. General requirements for the issuance of specific licenses.** Each application for a specific license shall be approved only if the application meets the requirements of these regulations.

(a) Each applicant shall be required to be qualified by reason of training and experience to use the material in question for the purpose requested, in accordance with these regulations, and in a manner that will protect public health and safety and the environment.

(b) The proposed equipment, facilities, and procedures used by each applicant shall protect public health and safety and the environment.

(c) ~~A specific license shall be approved only if the secretary determines that the specific license will not be a detriment to the health and safety of the public.~~

(~~d~~) Each applicant shall meet the requirements ~~prescribed~~ in these regulations for the particular license sought.

(e) (d)(1) Each application for a license for commercial waste disposal, source material milling, or any other operation that the secretary determines will affect the environment shall meet the requirement specified in this paragraph. Each application shall include information that permits the secretary to weigh the environmental, economic, technical, and other benefits against the environmental costs and alternatives to ensure the protection of public health and safety and the environment.

(2) The approval of each application specified in paragraph ~~(e)(1)~~ (d)(1) shall be based upon the following:

(A) The information specified in paragraph ~~(e)(1)~~ (d)(1) and other information as necessary; and

(B) the ~~applicable portions of~~ information required by 10 CFR part 51, subpart A, § C.F.R. 51.45, as in effect on April 30, 1992.

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(f) (e) Each applicant shall be authorized to begin construction only after the issuance of the license. Commencement of construction before issuance of the license shall be grounds for denial of the license application. "Commencement of construction," as used in this regulation, shall mean any clearing of land, excavation, or other substantial action that would adversely affect the environment of a site.

(g) (f) Each applicant for a license, other than a renewal, shall describe in the application how the facility design and procedures for operation will minimize, to the extent practicable, contamination of the facility and the environment, facilitate eventual decommissioning, and minimize, to the extent practicable, the generation of radioactive waste.

(h) (g) Each licensee who manufactures a nationally tracked source shall assign a unique serial number to each nationally tracked source manufactured by the licensee. Each serial number shall be composed only of alphanumeric characters.

(h) Each licensee shall conduct operations to minimize the introduction of residual radioactivity into the facility out to the site boundary, including the subsurface, in accordance with the existing radiation protection requirements and radiological criteria for license termination in these regulations. (Authorized by and implementing K.S.A. 48-1607;

effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended Sept. 20, 1993; amended Nov. 1, 1996; amended Dec. 30, 2005; amended July 27, 2007; amended

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**28-35-180b. Financial assurance for decommissioning.** (a) Each applicant for a specific license authorizing the possession and use of unsealed radioactive material with a half-life greater than 120 days and in quantities exceeding  $10^5$  times the applicable quantities specified in K.A.R. 28-35-201 shall submit a decommissioning funding plan as described in ~~K.A.R. 28-35-180b(e)~~ subsection (e) of this regulation. Each applicant shall also submit the decommissioning funding plan if a combination of isotopes is involved and if  $R$  divided by  $10^5$  is greater than one, where  $R$  is defined here as the sum of the ratios of the quantity of each isotope to the applicable value specified in K.A.R. 28-35-201.

(b) Each applicant for a specific license authorizing the possession and use of radioactive material with a half-life greater than 120 days and in quantities specified in table I shall submit either of the following:

(1) A decommissioning funding plan as described in subsection (e); or

(2) a certification that financial assurance for decommissioning has been provided in the amount prescribed by table I, using one of the methods described in subsection (f). The certification may state that the appropriate assurance is to be obtained after the application has been approved and the license has been issued, but before the receipt of licensed material. If the applicant defers execution of the financial instrument required under subsection (f) until after the license has been issued, a signed original of the financial instrument shall be submitted to the department before the applicant receives the licensed material. If the applicant does not defer execution of the financial instrument required under subsection (f), the applicant shall submit to the department, as part of the certification, a signed original of the financial instrument.

(c) Each holder of a specific license that is a type specified in subsection (a) or (b) shall

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provide financial assurance for decommissioning in accordance with the following requirements:

(1) Each holder of a specific license that is a type specified in subsection (a) shall submit a decommissioning funding plan as specified in subsection (e) or a certification of financial assurance for decommissioning in an amount equal to at least \$1,125,000.00. Each licensee shall submit the plan or certification to the department in accordance with the criteria specified in this regulation. If the licensee submits a certification of financial assurance rather than a decommissioning funding plan, the licensee shall include a decommissioning funding plan in any application for license renewal.

(2) Each holder of a specific license that is a type specified in subsection (b) shall submit a decommissioning funding plan as specified in subsection (e) or a certification of financial assurance for decommissioning. Each licensee shall submit the plan or certification to the department, in accordance with the requirements specified in this regulation.

(d) The amounts of financial assurance required for decommissioning, by quantity of material, shall be those specified in table I.

Table I

Financial assurance for decommissioning by quantity of material

If the possession limit is greater than $10^4$ but less than or equal to $10^5$ times the applicable quantities specified in K.A.R. 28-35-201, in unsealed form . . . . .	\$1,125,000.00
For a combination of isotopes, in unsealed form, if R, as defined in subsection (a), divided by $10^4$ is greater than one, but R divided	

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by 10 <sup>5</sup> is equal to or less than one . . . . .	\$1,125,000.00
If the possession limit is greater than 10 <sup>3</sup> but less than or equal to 10 <sup>4</sup> times the applicable quantities specified in K.A.R. 28-35-201, in unsealed form . . . . .	\$225,000.00
For a combination of isotopes, in unsealed form, if R, as defined in subsection (a), divided by 10 <sup>3</sup> is greater than one, but R divided by 10 <sup>4</sup> is less than or equal to one . . . . .	\$225,000.00
If the possession limit is greater than 10 <sup>10</sup> times the applicable quantities specified in K.A.R. 28-35-201, in sealed sources or foils . . . . .	\$113,000.00
For a combination of isotopes, in sealed sources or foils, if R, as defined in subsection (a), divided by 10 <sup>10</sup> is greater than one . . . .	\$113,000.00

(e) Each decommissioning funding plan shall contain the following:

(1) A cost estimate for decommissioning; in an amount including the following:

(A) The cost of an independent contractor to perform all decommissioning activities;

(B) the cost of meeting the requirements for unrestricted use specified in K.A.R. 28-35-205. However, if the applicant or licensee can demonstrate the ability to meet the provisions of K.A.R. 28-35-205a, the cost estimate may be based on meeting the requirements in K.A.R. 28-35-205a;

(C) the volume of on-site subsurface material containing residual radioactivity that will require remediation to meet the requirements for license termination; and

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(D) a contingency factor;

(2) a description of the method of ensuring funds for decommissioning, selected from the methods specified in subsection (f) identification of and justification for using the key assumptions contained in the decommissioning cost estimate;

(3) a description of the means for periodically a description of the method of ensuring funds for decommissioning from subsection (f), including means for adjusting cost estimates and associated funding levels periodically over the life of the facility;

(4) a certification by the licensee that financial assurance for decommissioning has been provided in the amount of the cost estimate for decommissioning; and

(5) a signed original of the financial instrument obtained to satisfy the requirements specified in subsection (f); and

(6) at the time of license renewal and at intervals not longer than three years, the decommissioning funding plan with adjustments necessary to account for changes in costs and the extent of contamination. The amount of financial assurance shall not be reduced without first obtaining the approval of an updated decommissioning funding plan. The decommissioning funding plan shall update the information submitted with the original or prior approved plan and shall specifically consider the effect of the following events on decommissioning costs:

(A) Spills of radioactive material producing additional residual radioactivity in on-site subsurface material;

(B) waste inventory exceeding the amount previously estimated;

(C) waste disposal costs exceeding the amount previously estimated;

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(D) facility modifications;

(E) changes in authorized possession limits;

(F) actual remediation costs exceeding the previous cost estimate;

(G) on-site disposal; and

(H) use of a settling pond.

(f) Each licensee shall provide financial assurance for decommissioning by one or more of the following methods: :

(1) Prepayment. "Prepayment" shall mean the deposit of cash or liquid assets ~~that meet the following criteria:~~

~~(A) before the start of operation, are deposited into an a trust account acceptable to the secretary that is segregated from the licensee's assets and outside of the licensee's administrative control; and~~

~~(B) .~~ The deposit shall consist of an amount that is sufficient to pay decommissioning costs. The adequacy of the trust funds shall be based on an assumed annual rate of return of one percent on the funds deposited into the trust.

~~The prepayment may be in the form of a trust, escrow account, government fund, certificate of deposit, or deposit of government securities.~~

(2) A surety instrument, insurance policy, or other guarantee method. The licensee may use a surety instrument, insurance policy, or other similar means to guarantee that decommissioning costs will be paid. A surety instrument may be in the form of a surety bond, letter of credit, or line of credit. A parent company's guarantee of funds for decommissioning costs based on a financial

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test may be used if the guarantee and test meet the requirements of K.A.R. 28-35-203. A parent company's guarantee shall not be used in combination with other financial methods to meet the requirements in this regulation. A guarantee of funds by the applicant or licensee for decommissioning costs based on a financial test may be used if the guarantee and test meet the requirements of K.A.R. 28-35-203. A guarantee by the applicant or licensee shall not be used in combination with any other financial methods to meet the requirements in this regulation or in any situation in which a parent company of the applicant or licensee holds majority control of the voting stock of the company. Each surety instrument or insurance policy used to provide financial assurance for decommissioning shall contain the following requirements:

(A) The surety instrument or insurance policy shall be open-ended or, if written for a specified term, shall be renewed automatically, unless 90 days or more before the renewal date, the insurer notifies the department, the beneficiary, and the licensee of the insurer's intention not to renew. The surety instrument or insurance policy shall also provide that the full face amount will be paid to the beneficiary automatically before the expiration without proof of forfeiture if the licensee fails to provide a replacement that meets the requirements of this regulation within 30 days after receipt of notification of cancellation.

(B) The surety instrument or insurance policy shall be payable to an approved trust established for decommissioning costs. The trustee may include an appropriate state or federal agency or an entity that has the authority to act as a trustee and whose trust operations are regulated and examined by a federal or state agency.

(C) The surety instrument or insurance policy shall remain in effect until the license is

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terminated by the department.

(3) External sinking fund. A licensee may provide financial assurance for decommissioning through an external sinking fund in which deposits are made at least annually, coupled with a surety instrument or insurance policy. The value of the surety instrument or insurance policy may decrease by the amount accumulated in the sinking fund. "External sinking fund" shall mean a fund that meets both of the following conditions:

(A) Is established and maintained by setting aside funds periodically in an account segregated from the licensee's assets and outside the licensee's administrative control; and

(B) contains a total amount of funds sufficient to pay the decommissioning costs when termination of the operation is expected. An external sinking fund may be in the form of a trust, escrow account, government fund, certificate of deposit, or deposit of government securities. The surety or insurance provisions shall meet the requirements specified in this subsection.

(4) Statement of intent. Any federal, state, or local government licensee may submit a statement of intent containing a cost estimate for decommissioning or an amount based on table I of this regulation and indicating that funds for decommissioning will be obtained when necessary.

(g) Each person licensed under subsections (a) through ~~(g)~~ (f) shall keep records of all information that is relevant to the safe and effective decommissioning of the facility. The records shall be kept in an identified location until the license is terminated by the department. If records of relevant information are kept for other purposes, the licensee may refer to these records and the location of these records within the records kept pursuant to this subsection.

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(h) Each licensee shall maintain decommissioning records, which shall consist of the following information:

(1) Records of spills or other unusual occurrences involving the spread of contamination in and around the facility, equipment, or site. These records may be limited to records of instances in which contamination remains after any cleanup procedures or when there is reasonable likelihood that contaminants could have spread to inaccessible areas. These records shall include any known information identifying the nuclides, quantities, forms, and concentrations involved in the spill or occurrence;

(2) drawings of the following, both as originally built and, if applicable, as modified:

(A) The structures and equipment in restricted areas where radioactive materials are used or stored, or both; and

(B) the locations of possible inaccessible contamination. If the licensee refers to required drawings other than those kept pursuant to this regulation, the licensee shall not be required to index each relevant document individually. If drawings are not available, the licensee shall substitute available information concerning these areas and locations;

(3) a list of the following information, which shall be contained in a single document and updated every two years:

(A) All areas designated and formerly designated as restricted areas;

(B) all areas outside of restricted areas that require the documentation specified in this subsection;

(C) all areas outside of restricted areas where current and previous wastes have been

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buried and documented as specified in K.A.R. 28-35-227j; and

(D) all areas outside of restricted areas that contain material so that, if the license expired, the licensee would be required either to decontaminate the area to unrestricted release levels or to apply for approval for disposal as specified in K.A.R. 28-35-225a.

Those areas containing sealed sources only shall not be included in the list if the sources have not leaked, no contamination remains in the area after any leak, or the area contains only radioactive materials having half-lives of less than 65 days; and

(4) the following records:

(A) Records of the cost estimate performed for the decommissioning funding plan or records of the amount certified for decommissioning; and

(B) if either a funding plan or certification is used, records of the funding method used for assuring funds.

(i) Each applicant for a specific license shall make ~~arrangements for a long-term care fund pursuant to K.S.A. 48-1623, and amendments thereto~~ available a long-term care fund necessary to provide for the long-term surveillance and care of the radioactive material or waste. Each applicant for any of the following types of specific licenses shall establish the long-term care fund before the issuance of the license or before the termination of the license if the applicant chooses, ~~at the time of licensure, to provide~~ by providing a surety instrument in lieu of a long-term care fund:

(1) Waste-handling licenses;

(2) source material milling licenses; and

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(3) licenses for any facilities formerly licensed by the U.S. atomic energy commission or the nuclear regulatory commission (NRC), if required.

(j)(1) Each applicant shall agree to notify the department, in writing, immediately following the filing of a voluntary or involuntary petition for bankruptcy under any chapter of title 11, bankruptcy, of the United States code by or against any of the following:

(A) The licensee;

(B) any person controlling the licensee or listing the license or licensee as property of the estate; or

(C) any affiliate of the licensee.

(2) The bankruptcy notification shall indicate the following:

(A) The name of the bankruptcy court in which the petition for bankruptcy was filed; and

(B) the date on which the petition was filed. (Authorized by and implementing K.S.A.

48-1607; effective Dec. 30, 2005; amended March 18, 2011; amended

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**28-35-181h. Specific licenses to manufacture and distribute the devices specified in**

**K.A.R. 28-35-178b.** An application for a specific license to manufacture and distribute one or more of the devices specified in K.A.R. 28-35-178b shall not be approved unless the applicant meets the requirements of subsections (a) and (b) of this regulation in addition to meeting all of the additional applicable requirements specified in these regulations.

(a) Each applicant shall submit information about the design, manufacture, prototype testing, quality control, labels, proposed uses, installation, servicing, leak testing, operating and safety instructions, and potential hazards of the device to provide reasonable assurance that the following conditions are met:

(1) The device can be safely operated by individuals not having training in radiological protection;

(2) under ordinary conditions of handling, storage, and use of the device, the radioactive material contained in the device will not be released or inadvertently removed from the device, and it is unlikely that any individual will receive a dose in excess of 10 percent of the limits specified in K.A.R. 28-35-212a; and

(3) under accident conditions, including fire and explosion, associated with handling, storage, and use of the device, it is unlikely that any individual will receive an external radiation dose or dose commitment in excess of the following organ doses:

(A) Whole body; head and trunk; active blood-forming organs; gonads; or lens of eye 15 rems

(B) Hands and forearms; feet and ankles; localized areas of skin averaged over areas no larger than 1 square centimeter 200 rems

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(C) Other organs 50 rems.

(b)(1) Each device shall bear a durable, legible, clearly visible label or labels that contain, in clearly identified and separate statements, the following information:

(A) Instructions and precautions necessary to ensure safe installation, operation, and servicing of the device. Operating and service manuals may be identified in the label and used to provide this information;

(B) specification of whether or not leak testing or testing of any on-off mechanism and indicator is required. The information shall include the maximum allowable time intervals between tests and shall identify the radioactive material by isotope, quantity of radioactivity, and date that the quantity was determined; and

(C) the information required in one of the following statements, as appropriate, in the same or a substantially similar form:

(i) "The receipt, possession, use, and transfer of this device, model \_\_\_\_\_, serial no. \_\_\_\_\_, are subject to a general license or the equivalent and the regulations of the U.S. nuclear regulatory commission or a state with which the U.S. nuclear regulatory commission has entered into an agreement for the exercise of regulatory authority. This label shall be maintained on the device in a legible condition. Removal of this label is prohibited.

CAUTION—RADIOACTIVE MATERIAL

\_\_\_\_\_

(Name of manufacturer or distributor)"; or

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(ii) “The receipt, possession, use, and transfer of this device, model \_\_\_\_\_, serial no. \_\_\_\_\_, are subject to a general license or the equivalent, and the regulations of a licensing state. This label shall be maintained on the device in a legible condition. Removal of this label is prohibited.

CAUTION—RADIOACTIVE MATERIAL

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(Name of manufacturer or distributor)”

(2) The model, serial number, and name of the manufacturer or distributor may be omitted from the requirements specified in paragraphs (b)(1)(C)(i) and (ii) if the information is elsewhere specified in labeling affixed to the device.

(3) Each device having a separate source housing that provides the primary shielding for the source shall also bear, on the source housing, a durable label containing the device model number and serial number, the isotope and quantity, the words “Caution — Radioactive Material,” the radiation symbol described in part 4 of these regulations, and the name of the manufacturer or initial distributor.

(4) Each device containing at least 370 Mbq (10 mCi) of cesium-137, 3.7 Mbq (0.1 mCi) of strontium-90, 37 Mbq (1 mCi) of americium-241 or any other transuranic element based on the activity indicated on the label shall meet the following criteria:

(A)(i) Bear a permanent label affixed to the source housing if the source housing is separable, including the words “Caution — Radioactive Material”; or

(ii) bear a permanent label affixed to the device if the source housing is not separable,

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including the words “Caution — Radioactive Material”; and

(B) if practicable, bear the radiation symbol described in part 4 of these regulations.

(c) If the device is required to be tested at intervals longer than six months, either for proper operation of the on-off mechanism and indicator, if any, or for leakage of radioactive material, or for both, the applicant shall include in the application sufficient information to demonstrate that the longer interval is justified by the performance characteristics of the device or of similar devices and by design features that have a significant bearing on the probability or consequences of leakage of radioactive material from the device or failure of the on-off mechanism and indicator. In determining the acceptable interval for the test for leakage of radioactive material, the applicant shall address the following in the application:

- (1) The primary containment of the source capsule;
- (2) protection of the primary containment;
- (3) the methods of sealing the primary containment;
- (4) the containment construction materials;
- (5) the form of contained radioactive material;
- (6) the maximum temperature withstood during prototype tests;
- (7) the maximum pressure withstood during prototype tests;
- (8) the maximum quantity of contained radioactive material;
- (9) the radiotoxicity of contained radioactive material; and
- (10) any prior operating experience with identical devices or similarly designed and constructed devices.

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(d) If the general licensee under K.A.R. ~~28-35-181b~~ 28-35-178b, or under equivalent regulations of an agreement state, is authorized to install the device, collect the sample to be analyzed by a specific licensee for leakage of radioactive material, service the device, test the on-off mechanism and indicator, or remove the device, the applicant shall include in the application the written instructions to be followed by the general licensee, the estimated calendar-quarter doses associated with each operation, and the bases for the estimates. The submitted information shall demonstrate that performance of the specified operations by an individual untrained in radiological protection, in addition to other handling, storage, and use of devices under the general license, is unlikely to cause that individual to receive a dose in excess of 10 percent of the annual limits specified in part 4 of these regulations.

(e) Each device shall be listed on the nuclear regulatory commission's sealed source and device registry. (Authorized by and implementing K.S.A. 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended Dec. 30, 2005; amended P-\_\_\_\_\_.)

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**28-35-181i. Special license to manufacture, distribute, assemble, or repair luminous safety devices for use in aircraft.** ~~An application~~ Each applicant for a specific license to manufacture, assemble, or repair luminous safety devices containing tritium or promethium-147, for use in aircraft, and to distribute ~~such~~ these devices to persons generally licensed under K.A.R. ~~28-35-178e~~ 28-35-178d shall ~~not be approved unless the applicant meets~~ meet the requirements of sections 10 C.F.R. 32.53, 32.54, 32.55, and 32.56 and 32.101 of 10 CFR Part 32, as in effect on ~~May 31, 1984~~ December 2, 2015, which are hereby adopted by reference; except that wherever the term "commission" appears within the text of the federal regulations adopted by reference in this regulation, that term shall be replaced with the term "department." (Authorized by and implementing K.S.A. ~~1984 Supp. 48-1607~~; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended P-\_\_\_\_\_.)

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**28-35-181k. Specific licenses to manufacture and distribute ice detection devices.** ~~An application~~ Each applicant for a specific license to manufacture ice detection devices and to distribute those devices to persons generally licensed under K.A.R. 28-35-178g shall ~~not be~~ approved unless the applicant meets meet the requirements of ~~sections~~ 10 C.F.R. 32.61, and 32.62, and 32.103 of 10 CFR Part 32, as in effect on ~~May 31, 1984~~ December 2, 2015, which are hereby adopted by reference, except that wherever the term "commission" appears within 10 C.F.R. 32.61, that term shall be replaced with the term "department." (Authorized by and implementing K.S.A. ~~1984 Supp.~~ 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended P-\_\_\_\_\_.)

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**28-35-181m. Specific licenses to manufacture, prepare, or distribute radiopharmaceuticals containing radioactive material for medical use.** An application for a specific license to manufacture, prepare, or distribute radiopharmaceuticals containing radioactive material and used by persons as specified in part 6 of these regulations shall not be approved unless the applicant meets the requirements of this regulation and all other applicable requirements of these regulations.

(a) Each applicant shall meet the requirements ~~specified~~ in K.A.R. 28-35-180a.

(b) Each applicant shall submit evidence of either of the following:

(1) The radiopharmaceutical containing radioactive material is subject to the federal food, drug and cosmetic act or the public health service act and will be manufactured, labeled, and packaged in accordance with a new drug application (NDA) approved by the food and drug administration (FDA), a biologic product license issued by the FDA, or a “notice of claimed investigational exemption for a new drug” (IND) accepted by the FDA.

(2) The manufacture and distribution of the radiopharmaceutical containing radioactive material is not subject to the federal food, drug, and cosmetic act or the public health service act.

(c) Each applicant shall submit evidence of at least one of the following:

(1) The applicant is registered or licensed with the U.S. food and drug administration as a drug manufacturer.

(2) The applicant is registered or licensed with a state agency as a drug manufacturer.

(3) The applicant is licensed as a pharmacy by the state board of pharmacy.

(4) The applicant is operating as a nuclear pharmacy within a federal medical institution.

(5) The applicant is operating a positron emission tomography (PET) drug production

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facility.

(d) Each applicant shall submit the following information on the radionuclide:

(1) The chemical and physical form of the material;

(2) the packaging in which the radionuclide is shipped, including the maximum activity per package; and

(3) evidence that the shielding provided by the packaging of the radioactive material is appropriate for the safe handling and storage of radiopharmaceuticals by group licensees.

(e)(1) Each applicant shall submit a description of the following:

(A) A label that shall be affixed to each transport radiation shield, whether the shield is constructed of lead, glass, plastic, or other material, of a radioactive drug to be transferred for commercial distribution. The label shall include the following:

(i) The radiation symbol and the words “CAUTION — RADIOACTIVE MATERIAL” or “DANGER — RADIOACTIVE MATERIAL”;

(ii) the name of the radioactive drug and the abbreviation; and

(iii) the quantity of radioactivity at a specified date and time. For radioactive drugs with a half-life greater than 100 days, the time may be omitted; and

(B) a label that shall be affixed to each syringe, vial, or other container used to hold a radioactive drug to be transferred for commercial distribution. The label shall include the radiation symbol and the words “CAUTION — RADIOACTIVE MATERIAL” or “DANGER — RADIOACTIVE MATERIAL” and an identifier that ensures that the syringe, vial, or other container can be correlated with the information on the transport radiation shield label.

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(2) The labels, leaflets, or brochures required by this regulation shall be made in addition to the labeling required by the FDA. The labels, leaflets, or brochures may be separate from the FDA labeling, or with the approval of the FDA, the labeling may be combined with the labeling required by the FDA.

(f) All of the following shall apply to each licensee described in paragraph (c)(3) or (c)(4), or both:

(1) The licensee may prepare radioactive drugs for medical use, if each radioactive drug is prepared by either an authorized nuclear pharmacist, as specified in paragraphs (2) and (4) of this subsection, or an individual under the supervision of an authorized nuclear pharmacist.

(2) The licensee may allow a pharmacist to work as an authorized nuclear pharmacist if at least one of the following conditions is met:

~~(A) The pharmacist qualifies as an authorized nuclear pharmacist.~~

~~(B)~~ The pharmacist meets the requirements specified in 10 CFR C.F.R. 35.55(b) and 35.59 as adopted by reference in K.A.R. 28-35-264, and the licensee has received an approved license amendment identifying this individual as an authorized nuclear pharmacist.

~~(C)~~ (B) The pharmacist is designated as an authorized nuclear pharmacist in accordance with paragraph (4) of this subsection.

(3) The actions authorized in paragraphs (1) and (2) of this subsection shall be permitted in spite of more restrictive language in license conditions.

(4) The licensee may designate a pharmacist as an authorized nuclear pharmacist if ~~the individual is a nuclear pharmacist preparing radioactive drugs and identified as an "authorized~~

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user” on a nuclear pharmacy license issued under this part: at least one of the following conditions is met:

(A) The individual was a nuclear pharmacist preparing only radioactive drugs containing accelerator-produced radioactive material.

(B) The individual practiced at a government agency or federally recognized Indian tribe pharmacy before November 30, 2007 or at any other pharmacy before August 8, 2009.

(5) Each licensee shall provide a copy of the state pharmacy license or registration for an individual to work as an authorized nuclear pharmacist and one of the following documents to the department ~~no later than 30 days after the date that the licensee allows, pursuant to paragraphs (2)(A) and (2)(C) of this subsection, the individual to work as an authorized nuclear pharmacist:~~

~~(A) A copy of each individual’s certification by a specialty board whose certification process has been recognized as specified in 10 CFR 35.55(a), as adopted by reference in K.A.R. 28-35-264, the department or agreement state license, the permit issued by a licensee of broad scope, or nuclear regulatory commission master materials permittee; and~~

~~(B) a copy of the state pharmacy license or registration. The individual’s certification by a specialty board whose certification process has been recognized as specified in 10 C.F.R. 35.55(a), as adopted by reference in K.A.R. 28-35-264;~~

(B) a department, NRC, or agreement state license listing the individual as an authorized nuclear pharmacist;

(C) an NRC master materials licensee permit listing the individual as an authorized nuclear pharmacist;

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(D) a permit issued by a licensee of broad scope or an NRC master materials permittee or the authorization from a commercial nuclear pharmacy that is authorized to list its own authorized nuclear pharmacist; or

(E) documentation that only accelerator-produced radioactive materials were used in the practice of nuclear pharmacy at a government agency or federally recognized Indian tribe before November 30, 2007 or at all other locations of use before August 8, 2009, or an earlier date noticed by the NRC as permitted by 10 C.F.R. 35.13(b)(5).

(g) Each licensee shall possess and use instrumentation to measure the radioactivity of radioactive drugs. Each licensee shall have procedures for using the instrumentation. Each licensee shall measure, by direct measurement or by combination of measurements and calculations, the amount of radioactivity in dosages of alpha-, beta-, or photon-emitting radioactive drugs before transfer for commercial distribution. Each licensee shall meet the following requirements:

(1) Perform tests before initial use, periodically, and following repair on each instrument for accuracy, linearity, and geometry dependence, as appropriate for the use of the instrument, and make adjustments if necessary; and

(2) check each instrument for constancy and proper operation at the beginning of each day of use.

(h) Each application from a medical facility, an educational institution, or a federal facility to produce positron emission tomography (PET) radioactive drugs for noncommercial transfer to licensees within the applicant's consortium authorized for medical use under part 6 of

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these regulations or equivalent agreement state requirements shall include the following:

(1) A request for authorization for the production of PET radionuclides or evidence of an existing license issued under these regulations or equivalent NRC or agreement state requirements for a PET radionuclide production facility within the applicant's consortium from which the applicant receives PET radionuclides;

(2) evidence that the applicant is qualified to produce radioactive drugs for medical use by meeting the requirements of this regulation;

(3) the name of each individual authorized to prepare PET radioactive drugs if the applicant is a pharmacy and documentation that each individual meets the requirements of an authorized nuclear pharmacist; and

(4) the name of each PET radioactive drug for production and noncommercial distribution to the applicant's consortium, including the chemical and physical form of each drug.

(i) Nothing in these regulations shall exempt the licensee from the requirement to comply with applicable FDA requirements and other federal and state requirements governing radioactive drugs. (Authorized by and implementing K.S.A. 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended Dec. 30, 2005; amended July 27, 2007; amended March 18, 2011; amended P-\_\_\_\_\_.)

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**28-35-181o. Specific licenses to manufacture and distribute sources and devices for use as a calibration, transmission, or reference source or for certain medical uses.** (a) Each application for a specific license to manufacture and distribute sources and devices containing radioactive material to persons licensed as specified in K.A.R. 28-35-181d for use as a calibration, transmission, or reference source or for one or more of the uses listed in 10 ~~CFR~~ C.F.R. 35.400, 35.500, 35.600, and 35.1000, as adopted by reference in K.A.R. 28-35-264, shall include the following information regarding each type of source or device:

- (1) The radioactive material contained, its chemical and physical form, and amount;
- (2) details of design and construction of the source or device;
- (3) procedures for, and results of, prototype tests to demonstrate that the source or device will maintain its integrity under stresses likely to be encountered in normal use and in accidents;
- (4) for devices containing radioactive material, the radiation profile for a prototype device;
- (5) details of quality control procedures to ensure that the production sources and devices meet the standards of the design and prototype tests;
- (6) procedures and standards for calibrating sources and devices;
- (7) legend and methods for labeling sources and devices as to their radioactive content;
- (8) radiation safety instructions for handling and storing the source or device. These instructions shall be included on a durable label attached to the source or device. However, instructions that are too lengthy for the label may be summarized on the label and printed in detail on a brochure that is referenced on the label; and
- (9) the label that is to be affixed to the source or device or to the permanent storage

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container for the source or device. The label shall contain information on the radionuclide, quantity, and date of assay, and a statement that the source or device is licensed by the department for distribution to persons licensed under K.A.R. 28-35-181d or under an equivalent license of the nuclear regulatory commission (NRC) or an agreement state. Labeling for sources that do not require long-term storage may be on a leaflet or brochure that is to accompany the source; and  
(10) documentation that the source or device is listed on the nuclear regulatory commission's sealed source and device registry.

(b)(1) If the applicant wants to have the source or device required to be tested for leakage of radioactive material at intervals longer than six months, the applicant shall include in the application sufficient information to demonstrate that the longer interval is justified by performance characteristics of the source or device, or similar sources or devices, and by design features that have a significant bearing on the probability or consequences of leakage of radioactive material from the source.

(2) In determining the acceptable interval between tests for leakage of radioactive material, information that includes the following shall be considered by the secretary:

- (A) The nature of the primary containment;
- (B) the method for protection of the primary containment;
- (C) the method of sealing the containment;
- (D) containment construction materials;
- (E) the form of the contained radioactive material;
- (F) the maximum temperature withstood during prototype tests;

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(G) the maximum pressure withstood during prototype tests;

(H) the maximum quantity of contained radioactive material;

(I) the radiotoxicity of contained radioactive material; and

(J) the applicant's operating experience with identical sources or devices or with similarly designed and constructed sources or devices. (Authorized by and implementing K.S.A. 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended July 27, 2007; amended March 18, 2011; amended P-\_\_\_\_\_.)

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**28-35-181t. Requirements for license to initially transfer source material for use under the small quantities of source material general license.** (a) Each person submitting an application for a specific license to initially transfer source material for use in accordance with K.A.R. 28-35-177a, or equivalent regulations of an agreement state or the nuclear regulatory commission (NRC), shall meet the following requirements:

(1) Meet the general requirements specified in K.A.R. 28-35-190a; and

(2) provide information documenting that the NRC approves the methods for quality control, labeling, and providing safety instructions to recipients.

(b) Each person licensed under this regulation shall meet the following requirements:

(1) Label the immediate container of each quantity of source material with the type of source material, the quantity of source material, and the words “radioactive material”;

(2) ensure that the quantities and concentrations of source material are labeled and indicated in any transfer records;

(3) provide the following information to each person to whom source material is transferred for use under K.A.R. 28-35-177a or equivalent regulations of an agreement state or the NRC before the source material is transferred for the first time in each calendar year to each person:

(A) A copy of K.A.R. 28-35-177a and K.A.R. 28-35-190a or relevant equivalent regulations of an agreement state or the NRC; and

(B) appropriate radiation safety precautions and instructions relating to the handling, use, storage, and disposal of the material;

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(4) report transfers as follows, on or before January 31 of each year covering all transfers for the previous calendar year:

(A) File a report with the department. The report shall include the following information:

(i) The name, address, and license number of the person who transferred the source material;

(ii) the name and address of the general licensee to whom source material is distributed, a responsible agent by name or position, or both, the phone number of the general licensee to whom the material was sent, and the type, physical form, and quantity of source material transferred; and

(iii) the total quantity of each type and physical form of source material transferred in the reporting period to all generally licensed recipients; and

(B) file a report with each agreement state or the NRC if the transfer is to a person licensed by the NRC that identifies all persons operating under provisions equivalent to K.A.R. 28-35-177a to whom more than 50 grams (0.11 lb) of source material has been transferred within a single calendar quarter. The report shall include the following information:

(i) The name, address, and license number of the person who transferred the source material;

(ii) the name and address of the general licensee to whom source material was distributed, a responsible agent by name or position, or both, the phone number of the general licensee to whom the material was sent, and the type, physical form, and quantity of source material transferred; and

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(iii) the total quantity of each type and physical form of source material transferred in the reporting period to each generally licensed recipient within the agreement state; and

(5) maintain all information that supports the reports required by this subsection concerning each transfer to a general licensee for one year after the transfer is included in a report to the NRC or to an agreement state.

(c) If no transfers were made to any person generally licensed under K.A.R. 28-35-177a, under an equivalent agreement state, or under NRC provisions during the period specified in paragraph (B)(4) of this regulation, a report shall be submitted to the NRC indicating that no transfers were made. If no transfers have been made to any general licensee in a particular agreement state during the reporting period, this information shall be reported to the agreement state upon request of the agency. (Authorized by and implementing K.S.A. 48-1607; effective P- \_\_\_\_\_.)

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**28-35-184a. Specific conditions on all licenses.** (a) No license and no right under any license shall be assigned or otherwise transferred except as authorized under the act or these regulations and approved by the secretary in writing. Each request to assign or transfer a license shall include the following:

(1) The name and the technical and financial qualifications of the proposed transferee;

and

(2) the financial assurance for decommissioning information required by K.A.R. 28-35-180b.

(b) Each person authorized under these regulations shall confine the use and possession of the radioactive material licensed to the locations and purposes authorized in the license.

(c) No person shall introduce radioactive material into any product or material knowing or having reason to believe that the product or material will be transferred to a person exempt from these regulations under K.A.R. 28-35-192a, 28-35-192b, 28-35-192c, ~~28-35-192d~~, 28-35-192e, 28-35-192f, or 28-35-192g or the equivalent regulations of the ~~United States~~ nuclear regulatory commission (NRC) or an agreement state, except in accordance with a specific license issued under K.A.R. 28-35-181f or the general license issued under K.A.R. 28-35-194a.

(d) Each licensee shall file written notice with the secretary 30 days before vacating any facility when the licensee decides to permanently discontinue all activities involving licensed materials authorized in that facility under the license.

(e) Each licensee authorized under K.A.R. 28-35-181h to distribute devices to generally licensed persons shall perform the following:

(1) Report to the department all sales or transfers of those devices to persons

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generally licensed under K.A.R. 28-35-178b. The report shall identify each general licensee by name and address, the type of device transferred, and the quantity and type of radioactive material contained in the device. A report shall be submitted within 90 days of the sale or transfer; and

(2) furnish, to each general licensee to whom the licensee transfers any such device, a copy of the general license issued under K.A.R. 28-35-178b.

(f)(1) Each general licensee that is required by this part to register and each specific licensee shall notify the department, in writing, immediately following the filing of a voluntary or involuntary petition for bankruptcy under any chapter of title 11, bankruptcy, of the United States code by or against any of the following:

(A) The licensee;

(B) any person controlling the licensee or listing the license or licensee as property of the estate; or

(C) any affiliate of the licensee.

(2) The notification specified in paragraph (f)(1) shall indicate the following:

(A) The name of the bankruptcy court in which the petition for bankruptcy was filed;

and

(B) the date of the filing of the petition.

(g) Each portable gauge licensee shall use at least two independent physical controls that form tangible barriers to secure each portable gauge from unauthorized removal whenever the portable gauge is not under the control and constant surveillance of the licensee. (Authorized by and implementing K.S.A. 48-1607; effective, T-86-37, Dec. 11,

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1985; effective May 1, 1986; amended Dec. 30, 2005; amended July 27, 2007; amended

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**28-35-192a. Exemptions; source material.** (a) ~~Any~~ Each person ~~shall be exempt from these regulations to the extent the person~~ who only acquires, possesses, uses, or transfers source material in any chemical mixture, compound, solution, or alloy in which the source material, by weight, is less than 0.05 percent of the mixture, compound, solution, or alloy shall be exempt from these regulations.

(b) ~~Any~~ Each person ~~shall be exempt from these regulations to the extent the person~~ who only acquires, possesses, uses, or transfers unrefined and unprocessed ore containing source material and does not refine or process the ore shall be exempt from these regulations.

(c) ~~Any~~ Each person ~~shall be exempt from these regulations to the extent the person~~ who only acquires, possesses, uses, or transfers any of the following shall be exempt from the requirements for a license in part 3 of these regulations and the requirements of parts 4 and 10 of these regulations:

(1) Any quantities of thorium contained in any of the following:

(A) Incandescent gas mantles;

(B) vacuum tubes;

(C) welding rods;

(D) electric lamps for illuminating purposes, if each lamp does not contain more than 50 milligrams of thorium;

(E) germicidal lamps, sunlamps, and lamps for outdoor or industrial lighting, if each lamp does not contain more than two grams of thorium;

(F) rare earth metals and compounds, mixtures, and products containing not more than 0.25 percent thorium or uranium, or both, by weight; or

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(G) personnel neutron dosimeters, if each dosimeter does not contain more than 50 milligrams of thorium;

(2) source material contained in any of the following:

(A) Glazed ceramic tableware, if the glaze contains not more than 20 percent source material, by weight;

(B) glassware, containing not more than two percent of source material by weight or, for glassware manufactured before August 27, 2013, 10 percent of source material by weight. This exemption shall not include commercially manufactured glass brick, pane glass, ceramic tile or other glass, or ceramic used in construction; ~~and~~

(C) glass enamel or glass enamel frit that contains not more than 10 percent of source material, by weight, and that was imported or ordered for importation into the United States, or initially distributed by manufacturers in the United States, before July 25, 1983; or

(D) piezoelectric ceramic containing not more than two percent of source material by weight;

(3) photographic film, negatives, and prints containing uranium or thorium;

(4) any finished product or part of a product fabricated of, or containing, tungsten or magnesium-thorium alloys if the thorium content of the alloy does not exceed four percent, by weight. The exemption contained in this paragraph shall not be deemed to authorize the chemical, physical, or metallurgical treatment or processing of any product or part of a product;

(5) ~~uranium contained in counterweights installed in aircraft, rockets, projectiles or missiles or stored or handled in connection with installation or removal of these counterweights~~

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when:

~~(A) the counterweights are manufactured in accordance with the specifications contained in a specific license issued by the secretary, the United States nuclear regulatory commission or an agreement state, and when distribution by the licensee is authorized pursuant to this paragraph or an equivalent provision of the regulations of the United States nuclear regulatory commission or an agreement state;~~

~~(B) each counterweight has been impressed in a manner that is clearly legible through any plating or covering with the following legend: "DEPLETED URANIUM"; and~~

~~(C) each counterweight is durably and legibly labeled or marked with the identification of the manufacturer, and the statement: "UNAUTHORIZED ALTERATIONS PROHIBITED". The exemption contained in this paragraph shall not be deemed to authorize the chemical, physical, or metallurgical treatment or processing of any counterweights, other than repair or restoration of any plating or other covering;~~

~~(6) uranium used as shielding and constituting part of any shipping container. The uranium shielding shall be conspicuously and ~~legible~~ legibly impressed with the ~~legend words~~ "CAUTION—RADIOACTIVE SHIELDING—URANIUM" and shall be enclosed in ~~mild~~ steel containing no more than 0.25 percent carbon, or another equally fire-resistant metal, with a minimum wall thickness of one-eighth inch (3.2 mm);~~

~~(7) (6) thorium or uranium contained in finished optical lenses, if each lens does not contain more than 30 percent of thorium or uranium by weight or, if manufactured after August 27, 2013, 10 percent of thorium or uranium by weight. The exemption ~~contained~~ in this~~

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paragraph shall not be deemed to authorize either of the following:

(A) The shaping, grinding, or polishing of the lens or any manufacturing processes other than the assembly of the lens into optical systems and devices without any alteration of the lens;  
or

(B) the receipt, possession, use, or transfer of thorium or uranium contained in contact lenses, or in ~~spectacles~~ eyeglasses, or in eyepieces in binoculars or other optical instruments;

~~(8)~~ (7) uranium contained in detector heads for use in fire detection units, if each detector head contains not more than 0.005 microcurie of uranium; ~~and~~ or

~~(9)~~ (8) thorium contained in any finished aircraft engine part containing nickel-thoria alloy, if both of the following conditions are met:

(A) The thorium is dispersed in the nickel-thoria alloy in the form of finely divided thoria (thorium dioxide); and

(B) the thorium content in the nickel-thoria alloy does not exceed four percent by weight.

~~(d) The exemptions provided in this regulation shall not authorize the manufacture, processing or production of any of the products described in this regulation.~~

(1) Each person who acquires, possesses, uses, or transfers uranium contained in counterweights installed in aircraft, rockets, projectiles or missiles or stored or handled in connection with installation or removal of these counterweights, except counterweights manufactured before December 31, 1969 under a specific license issued by the atomic energy commission and impressed with the legend required by that license, shall be exempt from the requirements for a license in part 3 of these regulations and the requirements of parts 4 and

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10 of these regulations if both of the following conditions are met:

(A) Each counterweight has been impressed in a manner that is clearly legible through any plating or covering with the following words: "DEPLETED URANIUM."

(B) Each counterweight is durably and legibly labeled or marked with the identification of the manufacturer and the following words: "UNAUTHORIZED ALTERATIONS PROHIBITED."

(2) The exemption specified in this subsection shall not be deemed to authorize the chemical, physical, or metallurgical treatment or processing of any counterweights, other than repair or restoration of any plating or other covering.

(e)(1) No person shall initially transfer for sale or distribution a product containing source material to any persons exempt under subsections (c) and (d) or equivalent regulations of an agreement state, unless authorized by a license issued by the nuclear regulatory commission (NRC) to initially transfer the products for sale or distribution.

(2) Each person authorized by an agreement state to manufacture, process, or produce materials or products containing source material and each person who imports finished products or parts for sale or distribution shall be authorized by a license issued by the NRC for distribution only. (Authorized by and implementing K.S.A. 1984 Supp. 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended P-\_\_\_\_\_.)

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**28-35-192c. Exceptions; other radioactive material.** Except for persons who apply tritium, promethium-147, or radium to, or persons who incorporate tritium, promethium-147, or radium into, the products listed in this regulation, ~~any each person shall be exempt from these regulations to the extent that the person~~ who only acquires, possesses, uses, or transfers any of the following products shall be exempt from these regulations:

(a) Timepieces or hands or dials containing radium, or timepieces, hands, or dials containing not more than the following specified quantities of other radioactive materials:

(1) 25 millicuries of tritium per timepiece;

(2) 5 millicuries of tritium per hand;

(3) 15 millicuries of tritium per dial. Bezels, when used, shall be considered as part of the dial;

(4) 100 microcuries of promethium-147 per watch or 200 microcuries of promethium-147 per any other timepiece;

(5) 20 microcuries of promethium-147 per watch hand or 40 microcuries of promethium-147 per hand on other timepieces;

(6) 60 microcuries of promethium-147 per watch dial or 120 microcuries of promethium-147 per dial on other timepieces. Bezels, when used, shall be considered as part of the dial. The levels of radiation from hands and dials containing promethium-147 shall not exceed the following, when measured through 50 milligrams per square centimeter of absorber:

(A) For wrist watches, 0.1 millirad per hour at 10 centimeters from any surface;

(B) for pocket watches, 0.1 millirad per hour at one centimeter from any surface; and

(C) for any other timepiece, 0.2 millirad per hour at 10 centimeters from any surface;

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and

(7) for intact timepieces manufactured before November 30, 2007, 0.037 megabecquerel (1 microcurie) of radium-226 per timepiece;

(b) balances of precision containing not more than one millicurie of tritium per balance or not more than 0.5 millicurie of tritium per balance part manufactured before December 17, 2007;

(c) marine compasses containing not more than 750 millicuries of tritium gas and other marine navigational instruments containing not more than 250 millicuries of tritium gas manufactured before December 17, 2007;

(d) ionization chamber smoke detectors containing not more than one microcurie ( $\mu\text{Ci}$ ) of americium-241 per detector in the form of a foil and designed to protect life and property from fires;

(e) electron tubes. The levels of radiation from each electron tube containing radioactive material shall not exceed one millirad per hour at one centimeter from any surface when measured through seven milligrams per square centimeter of absorber. For purposes of this subsection, "electron tubes" shall include spark gap tubes, power tubes, gas tubes including glow lamps, receiving tubes, microwave tubes, indicator tubes, pickup tubes, radiation detection tubes, and any other completely sealed tube that is designed to conduct or control electrical currents. An electron tube shall not contain more than one of the following specified quantities of radioactive material:

(1) 150 millicuries of tritium per microwave receiver protector tube or 10 millicuries of tritium per any other electron tube;

(2) 1 microcurie cobalt-60;

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- (3) 5 microcuries nickel-63;
- (4) 30 microcuries krypton-85;
- (5) 5 microcuries cesium-137; or
- (6) 30 microcuries promethium-147; and

(f) ionizing radiation-measuring instruments containing, for purposes of internal calibration or standardization, sources of radioactive material. No source shall exceed the applicable quantity ~~set forth~~ specified in K.A.R. ~~28-35-197a~~ 28-35-197b. No single instrument shall contain more than 10 sources. For the purposes of this subsection, 0.05  $\mu$ Ci of Am-241 shall be considered an exempt quantity. (Authorized by and implementing K.S.A. 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; amended March 18, 2011; amended P-\_\_\_\_\_.)

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**28-35-192g. Exemptions; exempt quantities.** (a) Except as provided in subsections (c) through (e), each person who acquires, possesses, uses, owns, receives, or transfers radioactive material in individual quantities that do not exceed the applicable quantity specified in K.A.R. ~~28-35-197a~~ 28-35-197b shall be exempt from these regulations.

(b) Each person who possesses radioactive material received or acquired before January 1, 1972 under the general license then provided in K.A.R. 28-35-178a shall be exempt from these regulations to the extent that the person possesses, uses, owns, or transfers that radioactive material. This exemption shall not apply to radium-226.

(c) This regulation shall not authorize the production, packaging, repackaging, or transfer of radioactive material for purposes of commercial distribution, or the incorporation of radioactive material into products intended for commercial distribution.

(d) No person shall, for purposes of commercial distribution, transfer radioactive material in the individual quantities specified in K.A.R. ~~28-35-197a~~ 28-35-197b knowing, or having reason to believe, that those quantities of radioactive material will be transferred to a person exempt under this regulation or an equivalent regulation of the nuclear regulatory commission (NRC) or an agreement state, except in accordance with a specific license issued by the secretary under K.A.R. 28-35-181r, an equivalent regulation of the ~~nuclear regulatory commission~~ NRC, or an equivalent regulation of an agreement state.

(e) No person shall, for purposes of producing an increased radiation level, combine quantities of radioactive material covered by this exemption so that the aggregate quantity exceeds the individual quantities specified in K.A.R. ~~28-35-197a~~ 28-35-197b. (Authorized by and implementing K.S.A. 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986;

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amended March 18, 2011; amended P-\_\_\_\_\_.)

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**28-35-192h. Certain industrial devices.** (a) Except as specified in subsections (b) and (c), each person who receives, possesses, uses, transfers, owns, or acquires any industrial device containing by-product material designed and manufactured for either of the following purposes shall be exempt from these regulations:

(1) Detecting, measuring, gauging, or controlling thickness, density, level, interface location, radiation, leakage, or qualitative or quantitative chemical composition; or

(2) producing an ionized atmosphere if the industrial device is manufactured, processed, produced, or initially transferred in accordance with a specific license issued by the nuclear regulatory commission (NRC).

(b) Each person who manufactures, processes, produces, or initially transfers for sale or distribution any industrial device containing by-product material designed and manufactured for either of the following purposes shall be excluded from the exemption in subsection (a):

(1) Detecting, measuring, gauging, or controlling thickness, density, level, interface location, radiation, leakage, or qualitative or quantitative chemical composition; or

(2) producing an ionized atmosphere.

(c) The exemption in subsection (a) shall exclude any source not incorporated into an industrial device, including calibration and reference sources.

(d) Each person who manufactures, processes, produces, or initially transfers for sale or distribution any industrial device containing by-product material for use under subsection (a) shall apply for a license and a certificate of registration from the NRC. (Authorized by and implementing K.S.A. 48-1607; effective P-\_\_\_\_\_.)

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28-35-197a. (Authorized by and implementing K.S.A. 1984 Supp. 48-1607; effective, T-86-37, Dec. 11, 1985; effective May 1, 1986; revoked P-\_\_\_\_\_.)

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**28-35-197b. Schedule B; exempt quantities of radioactive material.** The provisions of 10 C.F.R. 30.71, as in effect on October 1, 2007, are hereby adopted by reference, except that the word "byproduct" shall be replaced with "radioactive." (Authorized by and implementing K.S.A. 48-1607; effective P-\_\_\_\_\_.)

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**28-35-205b. Alternate criteria for license termination.** ~~(a)~~ A license shall be terminated by the secretary using alternate criteria greater than the dose criteria specified in K.A.R. 28-35-205a~~(d)~~ only if the licensee provides all of the following information:

~~(1)~~ (a) Evidence that public health and safety and the environment would continue to be protected and that it is unlikely that the dose from all manmade sources combined, other than medical, could be more than the limit of one millisievert per year or 100 mrem per year specified in part 4 of these regulations, by submitting an analysis of the possible sources of exposure;

~~(2)~~ (b) restrictions, to the extent practical, on site use according to the provisions of K.A.R. 28-35-205a to minimize exposure at the site;

~~(3)~~ (c) evidence that doses have been reduced to ALARA levels, taking into consideration any detriment, including any traffic accidents that could result from decontamination and waste disposal; ~~and~~

~~(4)~~ (d) a decommissioning plan indicating the licensee's intent to decommission in accordance with this part and specifying that the licensee proposes to decommission by the use of alternate criteria. The licensee shall document in the decommissioning plan how the advice of individuals and institutions in the community who might be affected by the decommissioning has been sought and addressed, as appropriate, following analysis of that advice. In seeking this advice, the licensee shall provide for the following:

~~(A)~~ (1) Participation by representatives of a broad cross section of community interests who could be affected by the decommissioning;

~~(B)~~ (2) an opportunity for comprehensive, collective discussions of the issues by the participants represented; and

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~~(C)~~ (3) a publicly available summary of the results of all the discussions specified in paragraph ~~(4)(B)~~ of this subsection (d)(2), including a description of the individual viewpoints of the participants on the issues and the extent of agreement and disagreement on the issues among the participants; and

(e) sufficient financial assurance, as specified in K.A.R. 28-35-180b, to enable an independent third party, including a governmental custodian of a site, to assume and carry out the responsibilities for any necessary control and maintenance of the site.

~~(b) The use of alternate criteria to terminate a license may be approved by the secretary only after the secretary's consideration of the staff's recommendations that address any comments provided by federal, state, and local governments and any public comments submitted pursuant to K.A.R. 28-35-206. (Authorized by and implementing K.S.A. 48-1607; effective Dec. 30, 2005; amended P-\_\_\_\_\_.)~~

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**28-35-217b. General monitoring requirements.** (a) Each licensee or registrant shall make, or cause to be made, surveys of each area of use, including the subsurface, that meet the following requirements:

(1) Provide measurements or evaluations demonstrating compliance with these regulations; and

(2) are necessary under the circumstances to evaluate the following:

(A) Radiation and radiological contamination levels;

(B) concentrations or quantities of radioactive material; and

(C) the potential radiological hazards that could be present.

(b) Records from surveys describing the location and amount of subsurface residual radioactivity identified at the facility out to the site boundary shall be kept on file with records required for decommissioning.

(c) ~~The~~ Each licensee or registrant shall ensure that instruments and equipment used for quantitative radiation measurements; are calibrated at intervals not to exceed 12 months, for the type of radiation measured.

~~(d)~~ The Each licensee or registrant shall ensure that adequate precautions are taken to prevent a deceptive exposure of an individual-monitoring device.

(e) All personnel dosimeters, except for direct and indirect reading pocket ionization chambers and those dosimeters used to measure the dose to the extremities that require processing to determine the radiation dose and are used by licensees to comply with these regulations or with conditions specified in a license, shall be processed and evaluated by a dosimetry processor that meets the following requirements:

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(1) Holds current personnel dosimetry accreditation from the national voluntary laboratory accreditation program (NVLAP) of the national institute of standards and technology;  
and

(2) is accredited for the type of radiation or radiations included in the NVLAP program that most closely approximates the type of radiation or radiations for which the individual wearing the dosimeter is monitored. (Authorized by and implementing K.S.A. 1993 Supp. 48-1607; effective Oct. 17, 1994; amended P-\_\_\_\_\_.)

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**28-35-221a. Procedures for picking up, transporting, receiving, and opening packages.** (a)(1) Each licensee or registrant who expects to receive a package containing quantities of radioactive material in excess of the type A quantities specified in K.A.R. 28-35-221b shall meet one of the following requirements:

(A) If the package is to be delivered to the licensee's or registrant's facility by the carrier, ~~shall~~ make arrangements to receive the package when it is offered for delivery by the carrier; or

(B) if the package is to be picked up by the licensee or registrant at the carrier's terminal, ~~shall~~ make arrangements to receive notification from the carrier of the arrival of the package, at the time of arrival.

(2) Each licensee or registrant who picks up a package of radioactive material from a carrier's terminal shall pick up the package ~~expeditiously~~ upon receipt of notification from the carrier of ~~its~~ the arrival of the package.

(b) Each licensee or registrant shall ensure that external radiation levels around any package specified in subsection (a) and, if applicable, external radiation levels around the vehicle transporting the package do not exceed 200 millirems per hour (2 mSv/hr) at any point on the external surface of the package or vehicle at any time during transportation. The transport index shall not exceed 10.

(c)(1) For the purpose of this subsection, "exclusive use" shall have the meaning specified in 10 C.F.R. 71.4, dated January 1, 2015 and hereby adopted by reference.

(2) For each package specified in subsection (a) and transported in exclusive use, radiation levels external to the package may exceed the limits specified in subsection (d) but shall not exceed any of the following:

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(A) 200 millirems per hour (2 mSv/hr) on the accessible external surface of the package unless the following conditions are met, in which case the limit shall be 1,000 millirems per hour (10 mSv/hr):

(i) The shipment is made in a closed transport vehicle. For the purposes of this subsection, “closed transport vehicle” shall mean a vehicle or conveyance equipped with a securely attached exterior enclosure that, during normal transportation, restricts the access of unauthorized persons to the cargo space containing a package specified in subsection (a). The enclosure can be either temporary or permanent and, in the case of packaged materials, can be the see-through type that limits access from top, sides, and bottom;

(ii) the package is secured so that its position within the closed transport vehicle remains fixed during transportation; and

(iii) no loading or unloading operations occur between the beginning and end of the transportation;

(B) 200 millirems per hour (2 mSv/hr) at any point on the outer surface of the closed transport vehicle, including the upper and lower surfaces, or for a flatbed-style closed transport vehicle with a personnel barrier, at any point on the vertical planes projected from the outer edges of the closed transport vehicle, on the upper surface of the load, and on the lower external surface of the closed transport vehicle;

(C) 10 millirems per hour (0.1 mSv/hr) at any point two meters from the vertical planes represented by the outer lateral surfaces of the closed transport vehicle, or, in the case of a flatbed-style closed transport vehicle, at any point two meters from the vertical planes projected from the

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outer edges of the closed transport vehicle; or

(D) two millirems per hour (0.02 mSv/hr) in any normally occupied positions in the closed transport vehicle, except that this paragraph shall not apply to private motor carriers if each person occupying any of these positions in the closed transport vehicle is provided with a personnel-monitoring device and training in accordance with K.A.R. 28-35-333.

(d) Each licensee or registrant, upon receipt of a any package of radioactive material, shall monitor the external surfaces of each package labeled with the U.S. department of transportation radioactive white I, or radioactive yellow II or III labels, as specified in 49 ~~CFR~~ C.F.R. 172.403 and 172.436-440 ~~in effect January 1, 1993,~~ for radioactive contamination caused by leakage of the radioactive contents. Each licensee or registrant shall also monitor for radiation levels ~~on~~ of each package containing quantities of radioactive materials that are equal to or more than ~~or equal to~~ the type A quantity ~~defined~~ specified in K.A.R. 28-35-221b. Each licensee or registrant shall monitor each package known to contain radioactive materials for radioactive contamination and radiation levels if there is evidence of degradation of package integrity. The monitoring shall be performed as soon as practicable after receipt, but not later than three hours after the package is received at the licensee's facility if received during the licensee's normal working hours or three hours from the beginning of the next working day if received after normal working hours. The licensee or registrant shall immediately notify the final delivery carrier and, by telephone ~~and telegram, mailgram, or facsimile,~~ the department ~~when~~ under either of the following conditions:

(A) (1) Removable radioactive surface contamination exceeds ~~the limits of K.A.R. 28-35-~~

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~~221b table V of these regulations; or~~ the following maximum permissible limits:

<u>Contaminant</u>	<u>Maximum Permissible Limits</u>		
	<u>Bq/cm<sup>2</sup></u>	<u>uCi/cm<sup>2</sup></u>	<u>dpm/cm<sup>2</sup></u>
<u>Beta and gamma emitters and low-toxicity</u>			
<u>alpha emitters</u>	<u>4</u>	<u>10<sup>-4</sup></u>	<u>220</u>
<u>All other alpha-emitting radionuclides</u>	<u>0.4</u>	<u>10<sup>-5</sup></u>	<u>22</u>

~~(B) (2)~~ (2) External radiation levels exceed the limits of specified in K.A.R. 28-35-221b(e) and ~~(f)~~.

(e) Each licensee or registrant shall establish and maintain procedures for safely opening packages in which radioactive material is received and shall ~~assure~~ ensure that these procedures are followed and ~~that due consideration is given to special instructions~~ any special instructions are followed for the type of package being opened.

~~(d) (f)~~ (f) ~~Licensees~~ Each licensee or registrant ~~registrant~~ transferring special form sources in vehicles owned or operated by the licensee or registrant to and from a work site shall be exempt from the contamination monitoring requirements of ~~K.A.R. 28-35-221a, but this~~ regulation. However, the licensee or registrant shall not be exempt from the monitoring requirement in K.A.R. 28-35-221a this regulation for measuring radiation levels that ensures that the source is still properly lodged in its shield. (Authorized by and implementing K.S.A. 1993 ~~Supp.~~ 48-1607; effective, T-85-43, Dec. 19, 1984; effective May 1, 1985; amended Sept. 20, 1993; amended Oct. 17, 1994; amended P- \_\_\_\_\_.)

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28-35-221b. Appendix A; determination of  $A_1$ , and  $A_2$ , and B quantities. (a)

Single radionuclides.

(1) For a single radionuclide of known identity, the values of  $A_1$  and  $A_2$  shall be taken from Table I if listed there. The values  $A_1$  and  $A_2$  in Table I shall also be applicable for the radionuclide contained in ( $\alpha$ ,n) or ( $\gamma$ ,n) neutron sources.

(2) For any single radionuclide whose identity is known but which is not listed in Table I, the value of  $A_1$  and  $A_2$  shall be determined according to the following procedure:

(A) If the radionuclide emits only one type of radiation,  $A_1$  shall be determined according to the following method. For radionuclides emitting different kinds of radiation, the value of  $A_1$  shall be the most restrictive value of those determined for each kind of radiation. However, in either case,  $A_1$  shall be no more than 1000 curies (37 TBq). If a parent nuclide decays into a shorter lived daughter with a half life not greater than 10 days,  $A_1$  shall be calculated for both the parent and the daughter, and the more limiting of the two values shall be assigned to the parent nuclide.

(i) For gamma emitters,  $A_1$  shall be determined by the expression:

$$A_1 = \frac{9}{\Gamma} \text{ curies}$$

where  $\Gamma$  is the gamma ray constant, corresponding to the dose in roentgens per curie-hour at one meter, and the number nine results from the choice of one rem per hour at a distance of three meters as the reference dose equivalent rate.

(ii) For x ray emitters,  $A_1$  shall be determined by the atomic number of the nuclide:

for  $Z \leq 55$ ,  $A_1 = 1000 \text{ Ci (37 TBq)}$ ; and

for  $Z > 55$ ,  $A_1 = 200 \text{ Ci (7.4 TBq)}$

where  $Z$  is the atomic number of the nuclide.

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(iii) For beta emitters,  $A_1$  shall be determined by the maximum beta energy ( $E_{max}$ ) according to Table II: and

(iv) For alpha emitters,  $A_1$  shall be determined by the expression:

$$A_1 = 1000 A_3$$

where  $A_3$  is the value listed in Table III;

(B)  $A_2$  is the more restrictive of the following two values:

(i) The corresponding  $A_1$ ; and

(ii) The value  $A_3$  obtained from Table III.

(3) For any single radionuclide whose identity is unknown, the value of  $A_1$  shall be taken to be 2 Ci (74 GBq) and the value of  $A_2$  shall be taken to be 0.002 Ci (74 MBq).

However, if the atomic number of the radionuclide is known to be less than 82, the value of  $A_1$  shall be taken to be 10 Ci (370 GBq) and the value of  $A_2$  shall be taken to be 0.4 Ci (14.8 GBq).

(b) Mixtures of radionuclides, including radioactive decay chains.

(1) For mixed fission products, the following activity limit shall be assumed if a detailed analysis of the mixture is not carried out.

$$A_1 = 10 \text{ Ci (370 GBq)}$$

$$A_2 = 0.4 \text{ Ci (14.8 GBq)}$$

(2) A single radioactive decay chain shall be considered to be a radionuclide when the radionuclides are present in their naturally occurring proportions and no daughter nuclide has a half life either longer than ten days or longer than that of the parent nuclide. The activity to be taken into account and the  $A_1$  or  $A_2$  value from table I to be applied are those corresponding to the parent nuclide of that chain. When calculating  $A_1$  or  $A_2$  values, radiation

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emitted by daughters shall be considered. However, in the case of radioactive decay chains in which any daughter nuclide has a half life either longer than ten days or greater than that of the parent nuclide, the parent and daughter nuclides shall be considered to be mixtures of different nuclides.

(3) In the case of a mixture of different radionuclides, where the identity and activity of each radionuclide are known, the permissible activity of each radionuclide  $R_1, R_2, \dots, R_n$  is such that  $F_1 + F_2 + \dots + F_n$  is not greater than unity, where:

$$F_1 = \frac{\text{total activity of } R_1}{A_1(R_1)}$$

$$F_2 = \frac{\text{total activity of } R_2}{A_1(R_2)}$$

$$F_n = \frac{\text{total activity of } R_n}{A_1(R_n)} \quad \text{and}$$

$A_1 (R_1, R_2, \dots, R_n)$  is the value of  $A_1$  or  $A_2$  as appropriate for the nuclide  $R_1, R_2, \dots, R_n$ .

(4) When the identity of each radionuclide is known but the individual activities of some of the radionuclides are not known, the formula given in paragraph three shall be applied to establish the values of  $A_1$  or  $A_2$  as appropriate. All the radionuclides whose individual activities are not known (their total activity will, however, be known) shall be classed in a single group and the most restrictive value of  $A_1$  or  $A_2$  applicable to any one of them shall be used as the value of  $A_1$  or  $A_2$  in the denominator of the fraction.

(5) Where the identity of each radionuclide is known but the individual activity of none of the radionuclides is known, the most restrictive value of  $A_1$  or  $A_2$  applicable to any one of the radionuclides present shall be adopted as the applicable value.

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~~(6) When the identity of none of the nuclides is known, the value of  $A_1$  shall be taken to be 2 Ci (74 GBq) and the value of  $A_2$  shall be taken to be 0.002 Ci (74 MBq). However, if alpha emitters are known to be absent, the value of  $A_2$  shall be taken to be 0.4 Ci (14.8 GBq).~~

Table I  
 $A_1$  and  $A_2$  Values for Radionuclides  
 (See Footnotes at end of Table)

Symbol of radionuclide	Element and atomic number	$A_1$ (Ci)	$A_2$ (Ci)	Specific Activity (Ci/g)
Ac-227	Actinium (89)	1000	0.003	$7.2 \times 10^4$
Ac-228		10	4	$2.2 \times 10^6$
Aq-105	Silver (47)	40	40	$3.1 \times 10^4$
Aq-110m		7	7	$4.7 \times 10^3$
Aq-111		100	20	$1.6 \times 10^5$
Am-241	Americium (95)	8	0.008	3.2
Am-243		8	0.008	$1.9 \times 10^4$
Ar-37 (compressed or uncompressed)*	Argon (18)	1000	1000	$1.0 \times 10^5$
Ar-41 (uncompressed)*		20	20	$4.3 \times 10^7$
Ar-41 (compressed)*		1	1	$4.3 \times 10^7$
As-73	Arsenic (33)	1000	400	$2.4 \times 10^4$
As-74		20	20	$1.0 \times 10^5$
As-76		10	10	$1.6 \times 10^6$
As-77		300	20	$1.1 \times 10^6$
At-211	Astatine (85)	200	7	$2.1 \times 10^6$

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Au-193	Gold (79)	200	200	$9.3 \times 10^5$
Au-196		30	30	$1.2 \times 10^5$
Au-198		40	20	$2.5 \times 10^5$

Table I (Continued-2)

Symbol of radionuclide	Element and atomic number	$A_1$ (Ci)	$A_2$ (Ci)	Specific Activity (Ci/g)
AU-199		200	25	$2.1 \times 10^5$
Ba-131	Barium (56)	40	40	$8.7 \times 10^4$
Ba-133		40	40	$4.0 \times 10^2$
Ba-140		20	20	$7.3 \times 10^4$
Be-7	Beryllium (4)	300	300	$3.5 \times 10^5$
Bi-206	Bismuth (83)	5	5	$9.9 \times 10^4$
Bi-207		10	10	$2.2 \times 10^2$
Bi-210 (RaE)		100	4	$1.2 \times 10^5$
Bi-212		6	6	$1.5 \times 10^7$
Bk-249	Berkelium (97)	1000	1	$1.8 \times 10^3$
Br-77	Bromine (35)	70	25	$7.1 \times 10^5$
Br-82		6	6	$1.1 \times 10^6$
C-11	Carbon (6)	20	20	$8.4 \times 10^8$
C-14		1000	60	4.6
Ca-45	Calcium (20)	1000	25	$1.9 \times 10^4$
Ca-47		20	20	$5.9 \times 10^5$
Cd-109	Cadmium (48)	1000	70	$2.6 \times 10^3$

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Cd-115m		30	30	$2.6 \times 10^4$
Cd-115		80	20	$5.1 \times 10^5$
Ce-139	Cerium-(58)	100	100	$6.5 \times 10^3$
Ce-141		300	25	$2.8 \times 10^4$
Ce-143		60	20	$6.6 \times 10^5$
Ce-144		10	7	$3.2 \times 10^3$
Cf-249	California-(98)	2	0.002	3.1

Table I-(Continued-3)

Symbol of radionuclide	Element and atomic number	$A_1(\text{Ci})$	$A_2(\text{Ci})$	Specific Activity (Ci/g)
Cf-250		7	0.007	$1.3 \times 10^2$
Cf-252		2	0.009	$6.5 \times 10^2$
Cl-36	Chlorine-(17)	300	10	$3.2 \times 10^{-2}$
Cl-38		10	10	$1.3 \times 10^8$
Cm-242	Curium-(96)	200	0.2	$3.3 \times 10^3$
Cm-243		9	0.009	$4.2 \times 10^4$
Cm-244		10	0.01	$8.2 \times 10^4$
Cm-245		6	0.006	$1.0 \times 10^4$
Cm-246		6	0.006	$3.6 \times 10^4$
Co-56	Cobalt-(27)	5	5	$3.0 \times 10^4$
Co-57		90	90	$8.5 \times 10^3$
Co-58m		1000	1000	$5.9 \times 10^6$
Co-58		20	20	$3.1 \times 10^4$

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Co-60		7	7	$1.1 \times 10^3$
Cr-51	Chromium (24)	600	600	$9.2 \times 10^4$
Cs-129	Cesium (55)	40	40	$7.6 \times 10^5$
Cs-131		1000	1000	$1.0 \times 10^5$
Cs-134m		1000	10	$7.4 \times 10^6$
Cs-134		10	10	$1.2 \times 10^3$
Cs-135		1000	25	$8.8 \times 10^{-1}$
Cs-136		7	7	$7.4 \times 10^4$
Cs-137		30	10	$9.8 \times 10^4$
Cu-64	Copper (29)	80	25	$3.8 \times 10^6$
Cu-67		200	25	$7.9 \times 10^5$

Table I (Continued-4)

Symbol of radionuclide	Element and atomic number	$A_1(\text{Ci})$	$A_2(\text{Ci})$	Specific Activity (Ci/g)
Dy-165	Dysprosium (66)	100	20	$8.2 \times 10^6$
Dy-166		1000	200	$2.3 \times 10^5$
Er-169	Erbium (68)	1000	25	$8.2 \times 10^4$
Er-171		50	20	$2.4 \times 10^6$
Eu-152m	Europium (63)	30	30	$2.2 \times 10^6$
Eu-152		20	10	$1.9 \times 10^3$
Eu-154		10	5	$1.5 \times 10^2$
Eu-155		400	60	$1.4 \times 10^3$
F-18	Fluorine (9)	20	20	$9.3 \times 10^7$

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Fe-52	Iron (26)	5	5	$7.3 \times 10^6$
Fe-55		1000	1000	$2.2 \times 10^3$
Fe-59		10	10	$4.9 \times 10^4$
Ga-67	Gallium (31)	100	100	$6.0 \times 10^5$
Ga-68		10	20	$4.0 \times 10^7$
Ga-72		7	7	$3.1 \times 10^6$
Gd-153	Gadolinium (64)	200	100	$3.6 \times 10^3$
Gd-159		300	20	$1.1 \times 10^6$
Ge-68	Germanium (32)	20	10	$7.0 \times 10^3$
Ge-71		1000	1000	$1.6 \times 10^5$
H-3	Hydrogen (1) see T-Tritium			
Hf-181	Hafnium (72)	30	25	$1.6 \times 10^4$
Hg-197m	Mercury (80)	200	200	$6.6 \times 10^5$
Hg-197		200	200	$2.5 \times 10^5$

Table I (Continued-5)

Symbol of radionuclide	Element and atomic number	A <sub>1</sub> (Ci)	A <sub>2</sub> (Ci)	Specific Activity (Ci/g)
Hg-203		80	25	$1.4 \times 10^4$
Ho-166	Holmium (67)	30	30	$6.9 \times 10^5$
I-123	Iodine (53)	50	50	$1.9 \times 10^6$
I-125		1000	70	$1.7 \times 10^4$
I-126		40	10	$7.8 \times 10^4$
I-129		1000	2	$1.6 \times 10^4$

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I-131		40	10	$1.2 \times 10^5$
I-132		7	7	$1.1 \times 10^7$
I-133		30	10	$1.1 \times 10^6$
I-134		8	8	$2.7 \times 10^7$
I-135		10	10	$3.5 \times 10^6$
In-111	Indium (49)	30	25	$4.2 \times 10^5$
In-113m		60	60	$1.6 \times 10^7$
In-114m		30	20	$2.3 \times 10^4$
In-115m		100	20	$6.1 \times 10^6$
Ir-190	Iridium (77)	10	10	$6.2 \times 10^4$
Ir-192		20	10	$9.1 \times 10^3$
Ir-194		10	10	$8.5 \times 10^5$
K-42	Potassium (19)	10	10	$6.0 \times 10^6$
K-43		20	10	$3.3 \times 10^6$
Kr-85m (uncompressed)*	Krypton (36)	100	100	$8.4 \times 10^6$
Kr-85m (compressed)*		3	3	$8.4 \times 10^6$
Kr-85 (uncompressed)*		1000	1000	$4.0 \times 10^2$
Kr-85 (compressed)*		5	5	$4.0 \times 10^2$

Table I (Continued-6)

Symbol of radionuclide	Element and atomic number	$A_1(\text{Ci})$	$A_2(\text{Ci})$	Specific Activity (Ci/g)
Kr-87 (uncompressed)*		20	20	$2.8 \times 10^7$
Kr-87 (compressed)*		0.6	0.6	$2.8 \times 10^7$

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La-140	Lanthanum (57)	30	30	$5.6 \times 10^5$
Lu-177	Lutetium (71)	300	25	$1.1 \times 10^5$
MFP	Mixed Fission products	10	0.4	—
Mg-28	Magnesium (12)	6	6	$5.2 \times 10^6$
Mn-52	Manganese (25)	5	5	$4.4 \times 10^5$
Mn-54		20	20	$8.3 \times 10^3$
Mn-56		5	5	$2.2 \times 10^7$
Mo-99	Molybdenum (42)	100	20	$4.7 \times 10^5$
N-13	Nitrogen (7)	20	10	$1.5 \times 10^9$
Na-22	Sodium (11)	8	8	$6.3 \times 10^3$
Na-24		5	5	$8.7 \times 10^6$
Nb-93m	Niobium (41)	1000	200	$1.1 \times 10^3$
Nb-95		20	20	$3.9 \times 10^4$
Nb-97		20	20	$2.6 \times 10^7$
Nd-147	Neodymium (60)	100	20	$8.0 \times 10^4$
Nd-149		30	20	$1.1 \times 10^7$
Ni-59	Nickel (28)	1000	900	$8.1 \times 10^2$
Ni-63		1000	100	$4.6 \times 10^4$
Ni-65		10	10	$1.9 \times 10^7$
Np-237	Neptunium (93)	5	0.005	$6.9 \times 10^4$
Np-239		200	25	$2.3 \times 10^5$

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Table I (Continued 7)

Symbol of radionuclide	Element and atomic number	A <sub>1</sub> (Ci)	A <sub>2</sub> (Ci)	Specific Activity (Ci/g)
Os-185	Osmium (76)	20	20	$7.3 \times 10^3$
Os-191		600	200	$4.6 \times 10^4$
Os-191m		200	200	$1.2 \times 10^4$
Os-193		100	20	$5.3 \times 10^5$
P-32	Phosphorus (15)	30	30	$2.9 \times 10^5$
Pa-230	Protactinium (91)	20	0.8	$3.2 \times 10^4$
Pa-231		2	0.002	$4.5 \times 10^{-2}$
Pa-233		100	100	$2.1 \times 10^4$
Pb-201	Lead (82)	20	20	$1.7 \times 10^6$
Pb-210		100	0.2	$8.8 \times 10^4$
Pb-212		6	5	$1.4 \times 10^6$
Pd-103	Palladium (46)	1000	700	$7.5 \times 10^4$
Pd-109		100	20	$2.1 \times 10^6$
Pm-147	Promethium (61)	1000	25	$9.4 \times 10^2$
Pm-149		10	20	$4.2 \times 10^5$
Po-210	Polonium (84)	200	0.2	$4.5 \times 10^3$
Pr-142	Praseodymium (59)	10	10	$1.2 \times 10^4$
Pr-143		300	20	$6.6 \times 10^4$
Pt-191	Platinum (78)	100	100	$2.3 \times 10^5$
Pt-193m		200	200	$2.0 \times 10^5$

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Pt-197m		300	20	$1.2 \times 10^{-7}$
Pt-197		300	20	$8.8 \times 10^{-5}$
Pu-238	Plutonium (94)	3	0.003	$1.7 \times 10^1$
Pu-239		2	0.002	$6.2 \times 10^{-2}$

Table I (Continued 8)

Symbol of radionuclide	Element and atomic number	$A_1(Ci)$	$A_2(Ci)$	Specific Activity (Ci/g)
Pu-240		2	0.002	$2.3 \times 10^{-1}$
Pu-241		1000	0.1	$1.1 \times 10^2$
Pu-242		3	0.003	$3.9 \times 10^{-3}$
Ra-223	Radium (88)	50	0.2	$5.0 \times 10^4$
Ra-224		6	0.5	$1.6 \times 10^5$
Ra-226		10	0.05	1.0
Ra-228		10	0.05	$2.3 \times 10^2$
Rb-81	Rubidium (37)	30	24	$8.2 \times 10^6$
Rb-86		30	30	$8.1 \times 10^4$
Rb-87		Unlimited	Unlimited	$6.6 \times 10^{-3}$
Rb (natural)		Unlimited	Unlimited	$1.8 \times 10^{-3}$
Re-186	Rhenium (75)	100	20	$1.9 \times 10^5$
Re-187		Unlimited	Unlimited	$3.8 \times 10^{-3}$
Re-188		10	10	$1.0 \times 10^6$
Re (natural)		Unlimited	Unlimited	$2.4 \times 10^{-3}$
Rh-103m	Rhodium (45)	1000	1000	$3.2 \times 10^2$

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Rh-105		200	25	$8.2 \times 10^5$
Rn-222	Radon (86)	10	2	$1.5 \times 10^5$
Ru-97	Ruthenium (44)	80	80	$5.5 \times 10^5$
Ru-103		30	25	$3.2 \times 10^4$
Ru-105		20	20	$6.6 \times 10^6$
Ru-106		10	7	$3.4 \times 10^3$
S-35	Sulphur (16)	1000	60	$4.3 \times 10^4$
Sb-122	Antimony (51)	30	30	$3.9 \times 10^5$

Table I (Continued-9)

Symbol of radionuclide	Element and atomic number	A <sub>1</sub> (Ci)	A <sub>2</sub> (Ci)	Specific Activity (Ci/g)
Sb-124		5	5	$1.8 \times 10^4$
Sb-125		40	25	$1.4 \times 10^3$
Sc-46	Scandium (21)	8	8	$3.4 \times 10^4$
Sc-47		200	20	$8.2 \times 10^5$
Sc-48		5	5	$1.5 \times 10^6$
Se-75	Selenium (34)	40	40	$1.4 \times 10^4$
Si-31	Silicon (14)	100	20	$3.9 \times 10^7$
Sm-147	Samarium (62)	Unlimited	Unlimited	$2.0 \times 10^{-8}$
Sm-151		1000	90	$2.6 \times 10^4$
Sm-153		300	20	$4.4 \times 10^5$
Sn-113	Tin (50)	60	60	$1.0 \times 10^4$
Sn-119m		100	100	$4.4 \times 10^3$

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Sn-125		10	10	$1.1 \times 10$
Sr-85m	Strontium (38)	80	80	$3.2 \times 10^7$
Sr-85		30	30	$2.4 \times 10^4$
Sr-85m		50	50	$1.2 \times 10^7$
Sr-89		100	10	$2.9 \times 10^4$
Sr-90		10	0.4	$1.5 \times 10^2$
Sr-91		10	10	$3.6 \times 10^6$
Sr-92		10	10	$1.3 \times 10^7$
T (uncompressed)*	Tritium (1)	1000	1000	$9.7 \times 10^3$
T (compressed)*		1000	1000	$9.7 \times 10^3$
T (activated luminous paint)		1000	1000	$9.7 \times 10^3$

Table I (Continued 10)

Symbol of radionuclide	Element and atomic number	$A_1(\text{Ci})$	$A_2(\text{Ci})$	Specific Activity (Ci/g)
T (absorbed on solid carrier)		1000	1000	$9.7 \times 10^3$
T (tritiated water)		1000	1000	$9.7 \times 10^3$
T (other forms)		20	20	$9.7 \times 10^3$
Ta-182	Tantalum (73)	20	20	$6.2 \times 10^3$
Tb-160	Terbium (65)	20	10	$1.1 \times 10^4$
Te-96m	Technetium (43)	1000	1000	$3.8 \times 10^7$
Te-96		6	6	$3.2 \times 10^5$
Te-97m		1000	200	$1.5 \times 10^4$

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Te-97		1000	400	$1.4 \times 10^{-3}$
Te-99m		100	100	$5.2 \times 10^6$
Te-99		1000	25	$1.7 \times 10^{-3}$
Te-125m	Tellurium (52)	1000	100	$1.8 \times 10^4$
Te-127m		300	20	$4.0 \times 10^4$
Te-127		300	20	$2.6 \times 10^6$
Te-129m		30	10	$2.5 \times 10^4$
Te-129		100	20	$2.0 \times 10^7$
Te-131m		10	10	$8.0 \times 10^5$
Te-132		7	7	$3.1 \times 10^5$
Th-227	Thorium (90)	200	0.2	$3.2 \times 10^4$
Th-228		6	0.008	$8.3 \times 10^2$
Th-230		3	0.003	$1.9 \times 10^{-2}$
Th-231		1000	25	$5.3 \times 10^5$
Th-232		Unlimited	Unlimited	$1.1 \times 10^{-7}$

Table I (Continued-11)

Symbol of radionuclide	Element and atomic number	A <sub>1</sub> (Ci)	A <sub>2</sub> (Ci)	Specific Activity (Ci-g)
Th-234		10	10	$2.3 \times 10^4$
Th (natural)		Unlimited	Unlimited	$2.2 \times 10^{-7}$
Th (irradiated)**		—	—	—
Tl-200	Thallium (81)	20	20	$5.8 \times 10^5$
Tl-201		200	200	$2.2 \times 10^5$

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Tl-202		40	40	$5.4 \times 10^4$
Tl-204		300	10	$4.3 \times 10^2$
Tm-170	Thulium (69)	300	10	$6.0 \times 10^3$
Tm-171		1000	100	$1.1 \times 10^3$
U-230	Uranium (92)	100	0.1	$2.7 \times 10^4$
U-232		30	0.03	$2.1 \times 10^4$
U-233		100	0.1	$9.5 \times 10^{-3}$
U-234		100	0.1	$6.2 \times 10^{-3}$
U-235		100	0.2	$2.1 \times 10^{-6}$
U-236		200	0.2	$6.3 \times 10^{-5}$
U-238		Unlimited	Unlimited	$3.3 \times 10^{-7}$
U-(natural)		Unlimited	Unlimited	see Table IV)
U-(enriched) < 20%		Unlimited	Unlimited	see Table IV)
20% or greater		100	0.1	see Table IV)
U-(depleted)		Unlimited	Unlimited	see Table IV)
U-(irradiated)***		---	---	---
V-48	Vanadium (23)	6	6	$1.7 \times 10^5$
W-181	Tungsten (74)	200	100	$5.0 \times 10^3$
W-185		1000	25	$9.7 \times 10^{-3}$

Table I (Continued-12)

Symbol of radionuclide	Element and atomic number	A <sub>1</sub> (Ci)	A <sub>2</sub> (Ci)	Specific Activity (Ci/g)
W-187		40	20	$7.0 \times 10^5$

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Xe-127 (uncompressed)*	Xenon (54)	70	70	$2.8 \times 10^4$
Xe-127 (compressed)*		5	5	$2.8 \times 10^4$
Xe-131m (compressed)*		10	10	$1.0 \times 10^5$
Xe-131m (uncompressed)*		100	100	$1.0 \times 10^5$
Xe-133 (uncompressed)*		1000	1000	$1.9 \times 10^5$
Xe-133 (compressed)*		5	5	$1.9 \times 10^5$
Xe-135 (uncompressed)*		70	70	$2.5 \times 10^5$
Xe-135 (compressed)*		2	2	$2.5 \times 10^5$
Y-87	Yttrium (39)	20	20	$4.5 \times 10^4$
Y-90		10	10	$2.5 \times 10^5$
Y-91m		30	30	$4.1 \times 10^7$
Y-91		30	30	$2.5 \times 10^4$
Y-92		10	10	$9.5 \times 10^0$
Y-93		10	10	$3.2 \times 10^6$
Yb-169	Ytterbium (70)	80	80	$2.3 \times 10^5$
Yb-175		400	25	$1.8 \times 10^5$
Zn-65	Zinc (30)	30	30	$8.0 \times 10^3$
Zn-69m		40	20	$3.3 \times 10^6$
Zn-69		300	20	$5.3 \times 10^7$
Zr-93	Zirconium (40)	1000	200	$3.5 \times 10^3$
Zr-95		20	20	$2.1 \times 10^4$
Zr-97		20	20	$2.0 \times 10^6$

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- \* For the purpose of Table I, compressed gas means a gas at a pressure which exceeds the ambient atmospheric pressure at the location where the containment system was closed.
- \*\* The values of  $A_1$  and  $A_2$  must be calculated in accordance with the procedure specified in Appendix A, paragraph II 3, taking into account the activity of the fission products and of the uranium-233 in addition to that of the thorium.
- \*\*\* The values of  $A_1$  and  $A_2$  must be calculated in accordance with the procedure specified in Appendix A, paragraph II 3, taking into account the activity of the fission products and plutonium isotopes in addition to that of the uranium.

Table II  
Relationship Between  $A_1$  and  $E_{max}$  for the Beta Emitters

$E_{max}$ (MeV)	$A_1$ (Ci)
$< 0.5$	1000
$0.5 < 1.0$	300
$1.0 < 1.5$	100
$1.5 < 2.0$	30
$\geq 2.0$	10

Table III  
Relationship Between  $A_1$  and the Atomic Number of the Radionuclide

Atomic Number	$A_2$		
	Half life less than 1000 days	Half life 1000 days to $10^6$ years	Half life greater than $10^6$ years
1 to 81	3 Ci	0.05 Ci	3 Ci
82 and above	0.002 Ci	0.002 Ci	3 Ci

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Table IV  
Activity-Mass Relationships for Uranium-Thorium

Thorium and Uranium Enrichment* wt % U-235 present	Specific Activity	
	Ci/g	µCi
0.45	$5.0 \times 10^{-7}$	$2.0 \times 10^6$
0.72 (natural)	$7.06 \times 10^{-7}$	$1.42 \times 10^6$

Table IV (continued)

Thorium and Uranium Enrichment* wt % U-235 present	Specific Activity	
	Ci/g	µCi
1.0	$7.6 \times 10^{-7}$	$1.3 \times 10^6$
1.5	$1.0 \times 10^{-6}$	$1.0 \times 10^6$
5.0	$2.7 \times 10^{-6}$	$3.7 \times 10^5$
10.0	$4.8 \times 10^{-6}$	$2.1 \times 10^5$
20.0	$1.0 \times 10^{-5}$	$1.0 \times 10^5$
35.0	$2.0 \times 10^{-5}$	$5.0 \times 10^4$
50.0	$2.5 \times 10^{-5}$	$4.0 \times 10^4$
90.0	$5.8 \times 10^{-5}$	$1.7 \times 10^4$
93.0	$7.0 \times 10^{-5}$	$1.4 \times 10^4$
95.0	$9.1 \times 10^{-5}$	$1.1 \times 10^4$
Natural Thorium	$2.2 \times 10^{-7}$	$4.6 \times 10^6$

\* The figures for uranium include representative values for the activity of the uranium-234 which is concentrated during the enrichment process. The activity for thorium includes the equilibrium concentration of thorium-228.

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(c) Type B quantity shall mean a quantity of radioactive materials greater than a type A quantity.

(d) The level of removable contamination on the external surfaces of each package shall, when averaged over the surface wiped, not exceed the limits given in table V below at any time during transport. The level of removable radioactive contamination shall be determined by wiping an area of 300 square centimeters of the surface concerned with an absorbent material, using moderate pressure, and measuring the activity on the wiping material. Sufficient measurements shall be taken in the most appropriate locations to yield a representative assessment of the removable contamination levels. Only in the case of packages transported as exclusive use shipment by rail or highway, may the removable radioactive contamination exceed the levels prescribed in table V. In this case, the levels shall not exceed 10 times the levels prescribed in table V.

Table V

Removable External Radioactive Contamination Wipe Limits

Contaminant	$\mu\text{Ci}/\text{cm}^2$	Maximum Permissible
		Limits
Beta-gamma emitting radionuclides;		$\text{dpm}/\text{cm}^2$
all radionuclides with half-lives		
less than ten days; natural uranium;		
natural thorium; uranium-235;		
uranium-238; thorium-232; thorium-228		

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and thorium-230 when contained in

ores or physical concentrates .....  $10^{-5}$  22

All other alpha emitting

radionuclides .....  $10^{-6}$  2.2

(e) External radiation levels around the package and around the vehicle, if applicable, shall not exceed 200 millirems per hour (2 mSv/hr) at any point on the external surface of the package at any time during transportation. The transport index shall not exceed 10.

(f) For a package transported in exclusive use by rail, highway or water, radiation levels external to the package may exceed the limits specified in K.A.R. 28-35-221b(d) but shall not exceed any of the following:

(1) 200 millirems per hour (2 mSv/hr) on the accessible external surface of the package unless the following conditions are met, in which case the limit shall be 1000 millirem per hour (10 mSv/hr):

(A) The shipment is made in a closed transport vehicle;

(B) provisions are made to secure the package so that its position within the vehicle remains fixed during transportation; and

(C) there are no loading or unloading operations between the beginning and end of the transportation;

(2) 200 millirems per hour (2 mSv/hr) at any point on the outer surface of the vehicle, including the upper and lower surfaces, or, in the case of a flat-bed style vehicle, with a personnel barrier, at any point on the vertical planes projected from the outer edges of the vehicle, on the upper surface of the load, and on the lower external surface of the vehicle;

(3) 10 millirems per hour (0.1 mSv/hr) at any point 2 meters from the vertical planes

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represented by the outer lateral surfaces of the vehicle, or, in the case of a flat bed style vehicle, at any point 2 meters from the vertical planes projected from the outer edges of the vehicle; and

(4) ~~2 millirems per hour (0.02 mSv/hr) in any normally occupied positions of the vehicle, except that this provision shall not apply to private motor carriers when persons occupying these positions are provided with special health supervision personnel radiation exposure monitoring devices, and training in accordance with K.A.R. 28-35-333.~~ The provisions of 10 C.F.R. part 71, appendix A, as in effect on June 12, 2015, are hereby adopted by reference, with the changes specified in this regulation.

(a) Wherever the term “commission” appears within 10 C.F.R. part 71, appendix A, that term shall be replaced with the term “department.”

(b) In 10 C.F.R. part 71, appendix A, paragraph II(c) shall be replaced with the following text: “The licensee shall submit requests for prior approval, described under paragraphs II(a) and II(b) of this appendix, to the department.” (Authorized by and implementing K.S.A. ~~1993 Supp.~~ 48-1607; effective Sept. 20, 1993; amended Oct. 17, 1994; amended P-\_\_\_\_\_.)

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**28-35-230d.** (Authorized by and implementing K.S.A. 48-1607; effective Oct. 17, 1994;  
amended Dec. 30, 2005; revoked P-\_\_\_\_\_.)

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**28-35-264. General requirements.** The provisions of 10 ~~CFR~~ C.F.R. part 35, as in effect on ~~January 15, 2010~~ June 30, 2015, are hereby adopted by reference, with the changes specified in this regulation.

(a) For the purposes of part 6, “byproduct material” shall mean all radioactive material regulated by the department.

(b) All reports required by this regulation shall be submitted to the department.

(c) The following sections shall be deleted:

(1) ~~10-CFR~~ 35.1, “purpose and scope”;

(2) ~~10-CFR~~ 35.2, “definitions,” except that the definitions of the following terms shall be retained:

(A) “Authorized medical physicist”;

(B) “authorized nuclear pharmacist”;

(C) “authorized user”;

(D) “medical event”;

(E) “prescribed dose”; and

(F) “radiation safety officer”;

(3) ~~10-CFR~~ 35.8, “information collection requirements: OMB approval”;

(4) ~~10-CFR~~ 35.18, “license issuance”;

(5) ~~10-CFR~~ 35.19, “specific exemptions”;

(6) ~~10-CFR~~ 35.26(a)(1), “radiation protection program changes”;

(7) ~~10-CFR~~ 35.4001, “violations”; and

(8) ~~10-CFR~~ 35.4002, “criminal penalties.”

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(d) Wherever the following ~~CFR~~ C.F.R. references occur within 10 ~~CFR~~ C.F.R. part 35, these references shall be replaced with the specified references to regulations and parts in this article:

(1) “10 CFR 19.12” shall be replaced with “K.A.R. 28-35-333, ‘instructions to workers.’ ”

(2) “10 CFR part 20” shall be replaced with “part 4, ‘standards for protection against radiation.’ ”

(3) “10 CFR 20.1101” shall be replaced with “K.A.R. 28-35-211d, ‘radiation protection programs.’ ”

(4) “10 CFR 20.1301(a)(1) and 20.1301(c)” shall be replaced with “K.A.R. 28-35-214a.”

(5) “10 CFR 20.1501” shall be replaced with “K.A.R. 28-35-217b.”

(6) “10 CFR part 30” shall be replaced with “part 3, ‘licensing of sources of radiation.’ ”

(7) “10 CFR 32.72” shall be replaced with “K.A.R. 28-35-181m, ‘specific licenses to manufacture and distribute radiopharmaceuticals containing radioactive material for medical use under group licenses,’ and K.A.R. 28-35-181n, ‘specific licenses to manufacture and distribute generators or reagent kits for preparation of radiopharmaceuticals containing radioactive material.’ ”

(8) “10 CFR 32.74” shall be replaced with “K.A.R. 28-35-181o, ‘specific licenses to manufacture and distribute sources and devices for use as a calibration or reference source, or for certain medical uses.’ ”

(9) “10 CFR 33.13” shall be replaced with “K.A.R. 28-35-182b, ‘qualifications for a type

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A specific license of broad scope.’ ”

(e) Wherever the following terms occur within 10 ~~CFR~~ C.F.R. part 35, these terms shall be replaced with “department”:

- (1) “Commission”;
- (2) “NRC operation center”; and
- (3) “NRC regional office.”

(f) The following changes shall be made to the sections specified:

(1) ~~10-CFR 35.6(b)(1)~~ and (c)(1) shall be replaced with the following text:

“Obtain review and approval of the research as specified in 45 CFR 46.111, ‘criteria for IRB approval of research’; and”.

(2) ~~10-CFR 35.6(b)(2)~~ and (c)(2) shall be replaced with the following text:

“Obtain informed consent from the human research subject as specified in 45 CFR 46.116, ‘general requirements for informed consent.’ ”

(3) ~~10-CFR 35.10~~, subsection (a) shall be deleted.

(4) In ~~10-CFR 35.10(d)~~, the date “October 24, 2002” shall be replaced with “the effective date of these regulations, ;” and in ~~10-CFR 35.10(b) and (c)~~, the date “October 25, 2005” shall be replaced with “two years from the effective date of these regulations.”

(5) ~~10-CFR 35.12(b)(1)~~ and ~~(e)(1)(i)~~ shall be replaced with the following text:

“submitting a form specified by the department that includes the facility diagram, equipment, and training and experience qualifications of the radiation safety officer, authorized users, authorized physicists, and authorized pharmacists.”

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(6) 35.12(c)(1)(i) shall be replaced with the following text: “a form specified by the department that includes the facility diagram, equipment, and training and experience qualifications of the radiation safety officer, authorized users, authorized physicists, and authorized pharmacists.”

(7) In ~~10-CFR~~ 35.57(a)(1) and (b)(1), the date “October 24, 2002” shall be replaced with “the effective date of these regulations.”

(7) (8) In ~~10-CFR~~ 35.57(a)(2) and (b)(2), the date “April 29, 2005” shall be replaced with “the effective date of these regulations.”

(8) (9) In ~~10-CFR~~ 35.432(a), the date “October 24, 2002” shall be replaced with “the effective date of these regulations.”

(9) (10) In ~~10-CFR~~ 35.3045, the footnote shall be deleted, and in subsection (a) the words “or any radiation-producing device” shall be added before the words “results in.”

(10) (11) ~~10-CFR~~ 35.3047(d) shall be replaced with the following text: “The licensee shall submit a written report to the department within 15 days after discovery of a dose to the embryo or fetus, or nursing child that requires a report in paragraphs (a) or (b) in this section.”

(11) (12) In ~~10-CFR~~ 35.3067, the phrase “with the department” shall be inserted after the word “report” in the first sentence, and the second sentence shall be deleted. (Authorized by and implementing K.S.A. 48-1607; effective Dec. 30, 2005; amended March 18, 2011; amended P-\_\_\_\_\_.)

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**28-35-288. Special requirements and exemptions for enclosed radiography.** (a) Each licensee or registrant shall ensure that each system for enclosed radiography that is designed to allow the admittance of any individual meets the following requirements:

(1) Meets all applicable requirements of this part and K.A.R. 28-35-214a if the system is not a certified cabinet X-ray system;

(2) meets all applicable requirements of this part and has been certified by the U.S. food and drug administration (FDA) as compliant with the requirements ~~specified~~ in 21 CFR C.F.R. 1020.40, if the system is a certified cabinet X-ray system; and

(3) is evaluated, at intervals not to exceed one year, to ensure compliance with the applicable requirements specified in paragraphs (1) or (2) of this subsection. A record of each evaluation shall be maintained for two years after the evaluation.

(b) Each cabinet X-ray system designed to exclude any individual shall be exempt from the requirements of K.A.R. 28-35-276, K.A.R. 28-35-278, K.A.R. 28-35-281, K.A.R. 28-35-282, K.A.R. 28-35-283, K.A.R. 28-35-284, K.A.R. 28-35-285, K.A.R. 28-35-286, and K.A.R. 28-35-289, with the following exceptions:

(1) Operating personnel shall be provided with personnel-monitoring equipment as specified in K.A.R. 28-35-217a.

(2) A registrant shall not permit any individual to operate a cabinet x-ray system until that individual has received a copy of and instruction in the operating procedures for the unit and has demonstrated competence in its use. A record that demonstrates compliance with this paragraph shall be maintained for inspection by the department until disposition is authorized by the department.

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(3) A test for proper operation of each high-radiation area control device or alarm system, where applicable, shall be conducted and recorded as specified in K.A.R. ~~28-35-288(e)~~ 28-35-288.

(c) Each permanent radiographic installation having any high-radiation area entrance control of the type specified in K.A.R. 28-35-219a shall also meet the following requirements:

(1) Each entrance that is used for personnel access to the high-radiation area in a permanent radiographic installation shall have both a visible and an audible warning signal to warn of the presence of radiation.

(2) The visible signal shall be activated by radiation whenever the source is exposed. The audible signal shall be activated if an attempt is made to enter the installation while the source is exposed.

~~(e)~~ (d) The control device or alarm system shall be tested for proper operation at the beginning of each period of use. A record of each test shall be prepared quarterly or before the first use after the end of the quarter. Each record shall be maintained for inspection by the department until the secretary authorizes disposal of the record. (Authorized by and implementing K.S.A. 48-1607; effective Jan. 1, 1970; amended, T-85-43, Dec. 19, 1984; amended May 1, 1985; amended Sept. 20, 1993; amended Dec. 30, 2005; amended P-\_\_\_\_\_.)

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**28-35-343. Storage precautions.** (a) Each source of radiation, except accelerators, shall be provided with a storage container and, if transported, a transport container. The same container may be used in both cases if the container meets the requirements for each use. The container shall be provided with a lock to prevent unauthorized removal of, or exposure to, the source of radiation.

(b) Each source of radiation shall be stored in a manner that minimizes danger from explosion or fire.

~~(c) Each licensee shall lock and physically secure the transport package containing licensed material in the transporting vehicle to prevent accidental loss, tampering, or unauthorized removal of the licensed material from the vehicle. (Authorized by and implementing K.S.A. 48-1607; effective Sept. 20, 1993; amended Dec. 30, 2005; amended P-\_\_\_\_\_.)~~

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**28-35-344. Transport precautions.** ~~Each transport container shall be physically secured to the transporting vehicle to prevent accidental loss, tampering, or unauthorized removal. Each licensee shall lock and physically secure each transport package containing licensed material in the transporting vehicle to prevent accidental loss, tampering, or unauthorized removal of the licensed material from the vehicle.~~ (Authorized by and implementing K.S.A. 1992 Supp. 48-1607; effective Sept. 20, 1993; amended P-\_\_\_\_\_.)

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**28-35-347. Quarterly In-person inventory.** Each licensee or registrant shall conduct a ~~physical~~ an in-person inventory to account for all sources of radiation once every ~~three~~ six months. A record of each inventory shall be maintained for two years from the date of the inventory for inspection by the department and shall include the quantities and kinds of sources of radiation, the location where sources of radiation are assigned, the date of the inventory, and the name of the individual conducting the inventory. (Authorized by and implementing K.S.A. ~~1992 Supp.~~ 48-1607; effective Sept. 20, 1993; amended P-\_\_\_\_\_.)

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**28-35-362. Notification of incidents, abandonment, and lost sources.** (a) ~~The~~ Each licensee shall notify the department of any incidents and any sources lost in other than downhole logging operations in accordance with K.A.R. 28-35-184b, 28-35-228a, 28-35-229a and 28-35-230a.

(b) Whenever a sealed source or device containing radioactive material is lodged downhole, the licensee shall:

(1) monitor at the surface for the presence of radioactive contamination with a radiation survey instrument or logging tool during logging tool recovery operations; ~~and~~ .

(2) (c) If the licensee knows or has reason to believe that a sealed source has been ruptured, the licensee shall notify the department immediately by telephone and subsequently, within 30 days, by confirmatory written report ~~if the licensee knows or has reason to believe that a sealed source has been ruptured~~. This written report shall identify the well or other location, describe the magnitude and extent of the escape of radioactive material, assess the consequences of the rupture, and explain efforts planned or being taken to mitigate these consequences.

(e) (d) If it becomes apparent that efforts to recover the radioactive source will not be successful, the licensee shall ~~comply with~~ meet the following requirements: :

(1) The licensee shall advise the ~~well-operator~~ well operator of the following requirements regarding the method of abandonment:

(A) The ~~well-operator~~ well operator shall immobilize and seal the radioactive source in place with a cement plug.

(B) The ~~well-operator~~ well operator shall set in place a whipstock or other deflection device.

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(C) The ~~well operator~~ well operator shall mount a permanent identification plaque at the surface of the well, containing the appropriate information required by this regulation.

(2) The licensee shall notify the department by telephone, giving the circumstances of the loss, and request approval of the proposed abandonment procedures.

(3) The licensee shall file a written report with the department within 30 days of the abandonment, ~~setting forth~~ providing the following information:

(A) The date of occurrence and a brief description of attempts to recover the source;

(B) a description of the radioactive source involved, including the radionuclide, quantity, and chemical and physical form;

(C) a description of the surface location and identification of the well;

(D) the results of efforts to immobilize and set the source in place;

(E) the depth of the radioactive source;

(F) the depth of the top of the cement plug;

(G) the depth of the well; and

(H) the information contained on the permanent identification plaque.

~~(d)~~ (e) Whenever a sealed source containing radioactive material is abandoned downhole, the licensee shall provide a permanent plaque ~~as described in K.A.R. 28-35-364 for posting~~ on the well or well-bore. The plaque shall meet the following requirements:

(1) Be constructed of long-lasting material, which may include stainless steel or Monel metal; and

(2) contain the following information engraved on its face:

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- (A) The word “CAUTION”;
- (B) the radiation symbol, without the conventional color requirement;
- (C) the date of abandonment;
- (D) the name of the well operator or well owner;
- (E) the well name and the well identification number or numbers or other designation;
- (F) a description of the sealed source or sources, by radionuclide and quantity of activity;
- (G) the source depth and the depth to the top of the plug; and
- (H) an appropriate warning ~~which~~ that, depending on the specific circumstances of that

abandonment, shall include one of the following:

- (i) “Do not drill below plug back depth”;
- (ii) “do not enlarge casing”; or
- (iii) “do not reenter the hole before contacting the Kansas department of health and

environment radiation control program”; and

(3) be ~~a minimum of~~ at least seven inches square. The word “CAUTION” shall be written in ½-inch letters and all other information shall be written in ¼-inch letters.

~~(e) (f) Each~~ If the licensee knows or has reason to believe that radioactive material has been lost in or to an underground potable water source, the licensee shall immediately notify the department by telephone; and subsequently, within 30 days, by confirming confirmatory letter; if the licensee knows or has reason to believe that radioactive material has been lost in or to an underground potable water source. The notice shall designate the well location and shall describe the magnitude and extent of loss of radioactive material, assess the consequences of such the

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loss, and explain efforts planned or being taken to mitigate these consequences. (Authorized by  
and implementing K.S.A. 48-1607; effective Sept. 20, 1993; amended Nov. 1, 1996; amended  
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**28-35-504. Advance notification of shipment of certain types of licensed or registered material.** (a)(1) As specified in subsections (b), (c), and (d) ~~of this regulation~~, each licensee shall provide advance notification to the governor, or the governor's designee, of each state of each shipment of licensed or registered material through or across the boundary of that governor's state. The licensee shall provide this advance notification before transporting, or delivering to a carrier for transport, any licensed or registered material outside the confines of the licensee's facility or other place of use or storage.

(2) As specified in subsections (b), (c), and (d), each licensee shall provide advance notification to the Indian tribal official or tribal official of participating tribes referenced in subsection (c), or the official's designee, of the shipment of licensed material within or across the boundary of the tribe's reservation before the transport or delivery to a carrier for transport of licensed material outside the confines of the licensee's plant or other place of use or storage.

(b)(1) The advance notification specified in subsection (a) shall be required for each shipment of irradiated reactor fuel containing 100 grams or less in net weight of irradiated fuel, exclusive of cladding and any other structural or packaging material, that has a total external radiation dose rate in excess of 100 rems per hour at a distance of three feet from any accessible surface without intervening shielding.

(2) The advance notification specified in subsection (a) shall also be required for each shipment of licensed or registered material, other than irradiated fuel, meeting all of the following conditions:

(A) The licensed or registered material is required to be shipped in a type B package for transportation as specified in this part.

(B) The licensed or registered material is being transported to or across a state boundary en route to a disposal facility or to a collection point for transport to a disposal

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facility.

(C) The quantity of licensed or registered material in a single package exceeds the smaller of the following:

(i) 3,000 times the  $A_1$  value of the radionuclides as specified in K.A.R. 28-35-221b for special form radioactive material or 3,000 times the  $A_2$  value of the radionuclides as specified in K.A.R. 28-35-221b for normal form radioactive material; and

(ii) 1,000 TBq (27,000 Ci).

(c) The notification specified in subsection (b) shall meet the following requirements:

(1) The notification shall be submitted, in writing, to the office of each appropriate governor or governor's designee and each appropriate Indian tribal official and to the director of the division of nuclear security in the office of nuclear security and incident response. ~~A list of names and addresses for the governor's designees can be obtained from one of the following sources:~~

~~(A) A list is published by the NRC annually in the federal register on or about June 30.~~

~~(B) A list is available on request from the director of the office of state programs at the U.S. NRC.~~

(2) Each notification delivered by mail shall be postmarked at least seven days before the beginning of the seven-day period during which departure of the shipment is estimated by the licensee to occur.

(3) Each notification delivered by any means other than mail shall reach the office of each governor or governor's designee and each appropriate Indian tribal official at least four days before the beginning of the seven-day period during which departure of the shipment is estimated by the licensee to occur.

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(4) Each licensee shall retain a copy of the notification as a record for three years.

(d) Each advance notification of any shipment of irradiated reactor fuel or nuclear waste shall contain the following information:

(1) The name, address, and telephone number of the shipper, carrier, and receiver of the irradiated reactor fuel or nuclear waste shipment;

(2) a description of the irradiated reactor fuel or nuclear waste contained in the shipment, as specified in the regulations of the ~~U.S. DOT~~ United States department of transportation (USDOT) in 49 ~~CFR~~ C.F.R. 172.202 and 172.203(d);

(3) a shipment schedule, which shall include the following information:

(A) The point of origin of the shipment and a specification of the seven-day period during which departure of the shipment is estimated by the licensee to occur;

(B) a specification of the seven-day period during which arrival of the shipment at the state boundaries is estimated by the licensee to occur; and

(C) the destination of the shipment and a specification of the seven-day period during which arrival of the shipment at the destination is estimated by the licensee to occur; and

(4) the name of a contact person, including a telephone number, for current shipment information.

(e) If any licensee finds out that the shipment schedule previously furnished to any governor, ~~or~~ governor's designee, or Indian tribal official in accordance with this regulation will not be met, that licensee shall perform the following:

(1) Telephone a responsible individual in the office of the governor or governor's designee or the Indian tribal official as soon as practical after the licensee has found out that the shipment schedule will not be met and inform that individual of the revised schedule; and

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(2) maintain a record of the name of the responsible individual contacted and the date of this contact for three years.

(f) Each licensee who cancels an irradiated reactor fuel or nuclear waste shipment for which advance notification has been sent shall send a cancellation notice to the governor of each state; or the governor's designee; or to the Indian tribal official who was previously notified and to the director of the division of nuclear security in the office of nuclear security and incident response. The licensee shall state in the notice that the notice is a cancellation and shall identify the advance notification that is being canceled. The licensee shall retain a copy of the notice as a record for three years. (Authorized by and implementing K.S.A. 48-1607; effective Dec. 30, 2005; amended P-\_\_\_\_\_.)

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**28-35-700. General requirements.** The provisions of 10 C.F.R. part 37, 78 fed. reg. 17007-17020 (2013), as in effect on May 20, 2013, are hereby adopted by reference, with the changes specified in this regulation.

(a) The following sections or portions of sections in 10 C.F.R. part 37 shall be deleted:

- (1) 37.1;
- (2) 37.3;
- (3) 37.7;
- (4) 37.9;
- (5) 37.11(a) and (b);
- (6) 37.13;
- (7) 37.43(d)(9);
- (8) in 37.81(g), the third sentence;
- (9) 37.105;
- (10) 37.107; and
- (11) 37.109.

(b) In 10 C.F.R. 37.5, the following terms and the definition of each of these terms shall be deleted:

- (1) "Act";
- (2) "agreement state";
- (3) "becquerel";
- (4) "byproduct material";
- (5) "commission";
- (6) "curie";

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- (7) "government agency";
- (8) "license";
- (9) "lost or missing licensed material";
- (10) "person";
- (11) "state"; and
- (12) "United States."

(c) Wherever the following words and phrases occur within the portions of 10 C.F.R. part 37 adopted in this regulation, these words and phrases shall be replaced with "department":

- (1) "Appropriate NRC regional office listed in §30.6(a)(2) of this chapter";
- (2) "Commission," except secs. 37.5, 37.27(a) and (c), 37.29(a) and 37.71;
- (3) "NRC," except secs. 37.25(b)(2), 37.27(c), 37.29(a), and 37.71;
- (4) "NRC regional office specified in §30.6 of this chapter";
- (5) "NRC's Operations Center"; and
- (6) "NRC's Operations Center (301-816-5100)."

(d) The following changes shall be made wherever the following text occurs within the portions of 10 C.F.R. part 37 adopted in this regulation:

- (1) "Part 73 of this chapter" shall be replaced with "10 C.F.R. Part 73."
- (2) "71.97(b) of this chapter" and "71.97 of this chapter" shall be replaced with "K.A.R 28-35-504(b)."
- (3) "Governor's designee" shall be replaced with "division of emergency management of the office of the adjutant general." (Authorized by and implementing K.S.A. 48-1607; effective P- \_\_\_\_\_.)

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