

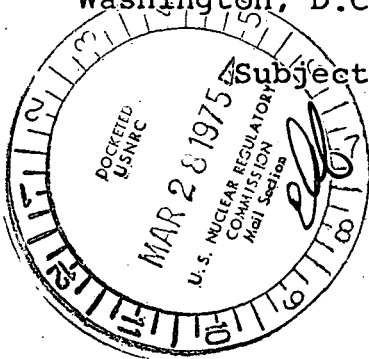
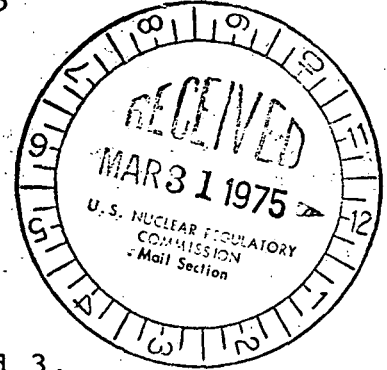


**Commonwealth Edison**  
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March 18, 1975

**REGULATORY DOCKET FILE COPY**

Mr. Edson G. Case  
 Acting Director  
 Office of Nuclear Reactor Regulation  
 U.S. Nuclear Regulatory Commission  
 Washington, D.C. 20555



**Subject:** Dresden Station Units 2 and 3,  
 Quad-Cities Station Units 1 and 2,  
 Dresden Station Special Report No. 39,  
 Quad-Cities Special Report No. 14,  
 Analysis of Hydrogen Generation and  
 Control in Primary Containment Following  
 Postulated Loss of Coolant Accident,  
 NRC Dkts. 50-237, 50-249, 50-254 and  
 50-265.

Dear Mr. Case:

Attached is a copy of the subject report which presents an analysis of generation and control of combustible gas in the containment. The purpose, the scope and the results of this review are thoroughly discussed in the report.

Your review of the subject report is requested. If you concur with the report's conclusion that the requirement for inerting the containment should be eliminated, the appropriate proposed amendments to Facility Operating Licenses DPR-19, DPR-25, DPR-29, and DPR-30 will be prepared and submitted for NRC approval.

The proposed license amendments are not included in this submittal, because the amount of metal-water reaction generated hydrogen calculated in this report using 10CFR50.46 criteria must be confirmed by the results of documented and approved analysis. Established techniques which are more precise than those used in Dresden Special Report No. 40 and Quad-Cities Special Report No. 15 (ECCS Analyses per 10CFR50.46) will be submitted to confirm the hydrogen generation calculated in the attached report. These confirmed calculations will not be

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Although the numerical guidelines of proposed Appendix I to Part 50 of Title 10 of the Code of Federal Regulations are not yet effective, the Regulatory staff is applying these guidelines for current light water reactor licensing actions. In general, the operations of a plant are controlled by the Technical Specifications appended to the operating license so that annual total body doses to individuals living near the plant should not exceed 5 millirem due to radioactive materials in liquid or gaseous effluents, and annual doses to any organ should not exceed 15 millirem due to radioiodine and particulate material in gaseous effluents. The potential doses are controlled by restrictions on the rate of liquid and gaseous radiological effluent releases and are further monitored through radiological environmental monitoring programs.