



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

April 6, 1976

Docket Nos. 50-237/249  
50-254/265

Commonwealth Edison Company  
ATTN: Mr. R. L. Bolger  
Assistant Vice President  
Post Office Box 767  
Chicago, Illinois 60690

Gentlemen:

RE: DRESDEN NUCLEAR POWER STATION UNITS 2 AND 3  
QUAD CITIES NUCLEAR POWER STATION UNITS 1 AND 2

The recent discovery of cracks in the feedwater nozzle blend radii at Quad Cities 1 and Pilgrim 1 Nuclear Power Stations, coupled with the previous discoveries at other stations, has amplified the need to determine the exact cause of the problem and thus undertake the proper short-term and long-term corrective actions. Your assistance is needed in the collection of pertinent data.

You are therefore requested, within 30 days of receipt of this letter, to provide response to the following questions:

1. If inspection(s) of feedwater nozzle blend radii has (have) been accomplished, supply the following information:
  - (a) Date of inspection(s)
  - (b) Inspection method used
  - (c) Inspection results and subsequent actions - number of cracks found; number of cracks which penetrated the cladding into the base metal; maximum depth of cracks; general location of worst cracking around each nozzle; method used to remove cracks; cross-sectional area of reinforcement removed at the worst grind-out location; subsequent inspections and results.
  - (d) If inspection report(s) has (have) been previously submitted to the NRC containing the desired information of (a), (b), and (c), provide reference to the report(s) in lieu of the information in (a), (b), and (c).

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2. If no inspection of feedwater nozzle blend radii has been conducted, provide date inspection is planned and date of next planned refueling outage.
3. If inspection(s) and repair(s) has (have) been undertaken, provide date of next planned inspection.
4. Provide the number of startup/shutdown cycles to the date of the inspection, if already accomplished, and the number of cycles since such inspection. If no inspection has been conducted, provide the estimated number of cycles at the planned inspection date. A startup/shutdown cycle is defined as a power increase from zero and subsequent return to zero.
5. Describe actions taken or planned to minimize cold, intermittent feedwater flow to the reactor pressure vessel. Include changes in operating procedures and any redesign of feedwater heaters, pumps, flow control valves, piping, etc.
6. Describe reactor pressure vessel pressure-temperature limits for operating conditions and especially for the inservice-hydrostatic and leak tests.

One signed original and 39 copies of your answer will be necessary.

This request for generic information was approved by GAO under a blanket clearance number B-180225 (R0072); this clearance expires July 31, 1977.

Sincerely,

*Richard D. Silver* for

Dennis L. Ziemann, Chief  
 Operating Reactors Branch #2  
 Division of Operating Reactors

cc: see next page

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April 6, 1976

cc

Mr. Charles Whitmore  
President and Chairman  
Iowa-Illinois Gas and  
Electric Company  
206 East Second Avenue  
Davenport, Iowa 52801

Mr. John W. Rowe  
Isham, Lincoln & Beale  
Counselors at Law  
One First National Plaza, 42nd Floor  
Chicago, Illinois 60603

Anthony Z. Roisman, Esquire  
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