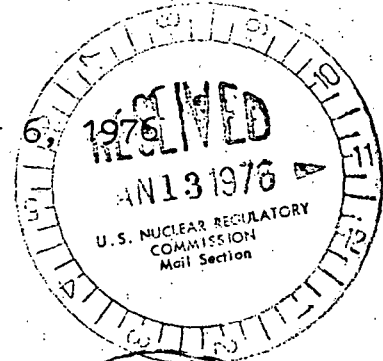




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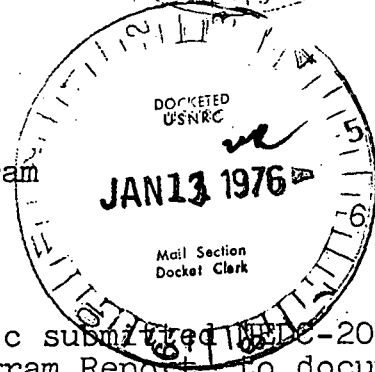
# REGULATORY DOCKET FILE COPY

January 6, 1976



Mr. Dennis L. Ziemann, Chief  
Operating Reactors - Branch 2  
Division of Reactor Licensing  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: Dresden Station Units 2 and 3  
Quad Cities Station Units 1 and 2  
Mark I Containment Short Term Program  
50-237, 50-249, 50-254, and 50-265



Dear Mr. Ziemann:

On October 6, 1975, General Electric submitted NEDC-20989, Mark I Containment Evaluation Short Term Program Report, to document work performed in response to the February 15 and April 17, 1975 requests for an evaluation of the Mark I Containment. As indicated in Volume I of the report, which was docketed by our letter of September 30, 1975, additional work not in the original Short Term Program was still underway and therefore not reported. A portion of this work has now been completed and was submitted along with a brief description of continuing work, on December 2, 1975, by letter from I.F. Stuart to R.S. Boyd. We have reviewed the information in the addendum which discusses pool swell loads on the MSR/V lines within the torus and the results of impact tests performed on a sample bellows unit. We find that this addendum does not alter the conclusions reached in the main body of the Short Term Final Report. Moreover, the MSR/V discharge line horizontal run for the Dresden and Quad Cities Units are below the water surface, and not subject to pool swell impact loads. In addition, the bellows on the subject units are outside the torus, and not subject to pool swell loads.

The other area where additional work is still underway is the evaluation of the Suppression Chamber Supports. General Electric is in the process of performing additional confirmatory tests on a 1/12th scale model segment to obtain data for identifying net downward and upward torus vertical loads, and confirmation of pool swell velocity with the effect of pressure over the pool surface considered. The additional work is necessary because the primary purpose of the initial 1/12th scale tests was to verify pool swell velocity. The results of the evaluation will be provided in Addendum 2 to NEDC-20989.

In the December 3 and 4 meeting in Bethesda at which Mark I Containment evaluation was reviewed, it was agreed that information

would be submitted concerning the continued operation of Mark I plants while the torus support system evaluation is completed. This basic evaluation is being conducted in response to the NRC's April 17, 1975 letter. We have previously responded to this letter with our letters dated May 7, 1975, July 31, 1975 and September 30, 1975. In addition, General Electric on our behalf has submitted the Short Term Program Final Report via Ivan F. Stuart's letter of October 6, 1975, and Addendum I via Mr. Stuart's letter of December 2, 1975. Work on the final evaluation of the torus support system is continuing and will be verbally reviewed with the NRC no later than January 22, 1976.

As discussed in the review meeting in Bethesda on December 4, there is significant technical basis for belief that the torus uplift load will be reduced. This has been discussed thoroughly in a General Electric letter report dated December 18, 1975, and which has been submitted on behalf of the Mark I utilities.

Based on our review and evaluation of the information presented by General Electric in the December 18 letter report, we have concluded that this information is applicable to Dresden Units 2 and 3, and Quad Cities Units 1 and 2. Considering the extremely low probability of a LOCA event occurring at our plant during the next six weeks and the high likelihood of a reduction of loads as a result of this evaluation, we have concluded that Dresden Units 2 and 3, and Quad Cities Units 1 and 2 can continue operation without danger to the public health and safety while the evaluation is continued.

As was discussed at the December 3-4 meeting, Addendum Number II to the Short Term Program Final Report will be issued approximately March 15, 1976, finalizing the results of this evaluation. We have not attempted to address your requests for additional information which were transmitted by the letters of November 4, 1975, and September 30, 1975, to Mr. R.L. Bolger. It is our understanding from Mr. T. Keenan, the chairman of the Mark I utility group, that no response to these questions from individual utilities is necessary at this time based on Mr. Keenan's discussions with your Mr. T. Paulson.

Very truly yours,



G.A. Abrell  
Nuclear Licensing Administrator  
Boiling Water Reactors