



Commonwealth Edison
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May 16, 1977

Mr. Edson G. Case, Deputy Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555



Subject: Dresden Station Units 2 and 3
Quad-Cities Station Units 1 and 2
Proposed Amendment to Facility
Operating License Nos. DPR-19, 25,
29, and 30
NRC Docket Nos. 50-237/249/254/265

Regulatory

File Cy.

Dear Mr. Case:

Pursuant to 10 CFR 50.59, Commonwealth Edison proposes to make amendments to Dresden Units 2 and 3 and Quad-Cities Units 1 and 2 Technical Specifications concerning reactor vessel metal surveillance program. The purpose of this change is to make the surveillance program consistent with 10 CFR 50, Appendix H.

The proposed change to Dresden Units 2 and 3 will require amending pages 88, 93, and adding pages 93A and 94A to both DPR-19 and 25. Section 4.6.B.3 should be changed to read:

"Neutron flux monitors and samples shall be installed in the reactor vessel adjacent to the vessel wall at the core midplane level. The monitor and sample program where possible conform to ASTM E185. The monitors and samples will be removed and tested as outlined in Table 4.6.2 to experimentally verify the calculated values of integrated neutron flux that are used to determine NDTT for Figure 4.6.1."

Bases: 3.6/4.6.B on page 93 should be amended to include the basis for the change. The additional basis will read:

"The withdrawal schedule in Table 4.6.2 is based on the three capsule surveillance program as defined in Section 11.C.3.a of 10 CFR 50 Appendix H. The accelerated capsule (Near Core Top Guide) are not required by Appendix H but will be tested to provide additional information on the vessel material.

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This surveillance program conforms to ASTM E-185-73 "Recommended Practice for Surveillance Tests for Nuclear Reactor Vessels" with one exception. The base metal specimens of the vessel were made with their longitudinal axes parallel to the principle rolling direction of the vessel plate."

Page 93A is being added because the above mentioned additional basis displaced part of the existing basis onto another page.

Page 94A is being added to accommodate "Table 4.6.2" which defines the sample withdrawal schedule.

The proposed change to Quad-Cities Units 1 and 2 will require amending pages 3.6/4.6-2, 3.6/4.6-9, and adding pages 3.6/4.6-9A and 3.6/4.6-21A to both DPR-29 and 30. Section 4.6.B.3 should be changed to read:

"Neutron flux monitors and samples shall be installed in the reactor vessel adjacent to the vessel wall at the core midplane level. The monitor and sample program shall as a minimum conform to ASTM E-185-66. The monitors and samples shall be removed and tested in accordance with the guidelines set forth in 10 CFR 50 Appendix H, to experimentally verify the calculated values of integrated neutron flux that are used to determine the NDTT for Figure 3.6-1."

Bases 3.6.B on page 3.6/4.6-9 should be amended to include the basis for the change. The additional basis will read the same as Dresden's. Page 3.6/4.6-9A is being added because the above mentioned additional basis displaced part of the existing basis onto another page. Page 3.6/4.6-21A is being added to accommodate "Table 4.6.2" which defines the sample withdrawal schedule.

The reactor vessel metal surveillance programs have been updated and reevaluated by Commonwealth Edison. As a result of this reevaluation we recommend new withdrawal schedules based on 10 CFR Appendix H. Section 11.B of 10 CFR 50 Appendix H requires that the surveillance program conform with ASTM E-185-73 "Recommended Practice for Surveillance Tests for Nuclear Reactor Vessels." ASTM E-185-73 specifies the location and orientation of the actual

Commonwealth Edison

Mr. Edson G. Case

- 3 -

May 16, 1977

test specimens from the plate material. All base-metal Charpy "V" notch specimens are to be cut transverse to the principle rolling, whereas actual specimens were cut parallel to the principle rolling direction. The direction of actual specimen cut is in accordance with ASTM E-185-66 since ASTM E-185-73 was not yet written when the specimens were cut. This is the only exception to the requirements of ASTM E-185-73. Therefore, the reference to ASTM E-185-66 remains unchanged in the Quad-Cities Technical Specification to allow this exception.

This proposed change does not increase the probability of any occurrence or the consequence of any accident or malfunction of equipment. The margin of safety, as defined in the basis for any technical specification is not reduced, because the intent of the technical specification is not changed.

Attachment A contains Dresden Units 2 and 3 changes and Attachment B contains Quad-Cities Units 1 and 2 changes.

These Technical Specification changes have received on-site and off-site review and approval. Please direct any additional questions to this office.

Three (3) signed originals and 57 copies are provided for your use.

Very truly yours,



R. L. Bolger
Assistant Vice President

SUBSCRIBED and SWORN to
before me this 24th day
of May, 1977.

Nancy M. Hallingworth
NOTARY PUBLIC