

Docket Nos. 50-237

50-249 ✓

50-254

and 50-265

DEC 12 1977

Commonwealth Edison Company

ATTN: Mr. R. L. Bolger

Assistant Vice President

P. O. Box 767

Chicago, Illinois 60690

Gentlemen:

RE: DRESDEN NUCLEAR POWER STATION UNITS 2 AND 3
QUAD-CITIES STATION UNITS 1 AND 2 -
SUPPRESSION POOL TEMPERATURE TRANSIENTS

As part of our continuing review of the effects of elevated suppression pool temperatures on safety-relief valve (SRV) discharge loads, we have recently transmitted a request for additional information to General Electric (GE) regarding topical report NEDE 21078P, "Test Results Employed by GE for BWR Containment and Vertical Vent Loads." Certain information which we require in order to complete our review of this report is of a plant-specific nature. Since you have adopted this topical report as part of your docketed material, we are transmitting to you the enclosed request for additional information.

You are requested to provide the information identified in the enclosure as soon as practical. Your projected submittal date for your response should be provided within ten days of your receipt of this letter.

The material contained in NEDE 21078P, the supplemental generic information requested from GE, and the plant-specific information requested in this letter will serve as part of the bases for our review of SRV loads in the Mark I Containment Long-Term Program (LTP). Since the referenced report has been classified as proprietary under our rules, and since we have determined that the requests contained in Part B of the enclosure are proprietary, they are, therefore, being withheld from public disclosure.

A-3

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We are aware that the Mark I Owners Group is developing a quencher-type SRV discharge device as part of the LTP. Although the quencher devices are believed to be less sensitive to elevated pool temperatures, it is our view that licensee responses to the enclosed request for additional information should be provided for each facility regardless of the type of discharge device that will be utilized upon completion of the LTP.

Sincerely,

Don K. Davis, Acting Chief
 Operating Reactors Branch #2
 Division of Operating Reactors

Enclosure:
 Request for Additional Information
 (Part A - Non-Proprietary)
 (Part B - Proprietary)

cc w/enclosure (Part A):
 See next page

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DATE >	12/ /77	12/ /77	12/ /77	12/ /77	

cc w/enclosure (Part A):
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REQUEST FOR ADDITIONAL INFORMATION
REGARDING SUPPRESSION
POOL TEMPERATURE TRANSIENTS

Part A: Non-Proprietary

1. Provide figures which depict the reactor pressure, safety/relief valve (SRV) discharge mass flux, and suppression pool bulk temperature versus time for the following events which are based on current Technical Specification limits:
 - (a) Stuck-open SRV during power operation assuming reactor scram at ten minutes after the suppression pool reaches a bulk pool temperature of 110 F and all RHR systems are operable.
 - (b) Same events as in (a) above with only one RHR train operable.
 - (c) Stuck-open SRV during hot standby assuming an initial 120 F bulk pool temperature and only one RHR train operable.
 - (d) Automatic Depressurization System (ADS) activated following a small line break assuming an initial 120 F bulk pool temperature and only one RHR train operable.
 - (e) Primary system is isolated and depressurized at a rate of 100 F per hour with an initial 120 F bulk pool temperature and only one RHR train operable.
2. Briefly describe the suppression pool temperature monitoring system at your facility and the relative location of the temperature sensors to the SRV discharge points.