UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 5, 2017

Mr. Mano Nazar
President and Chief Nuclear Officer
Nuclear Division
Florida Power & Light Company
Mail Stop: EX/JB
700 Universe Blvd.
Jupiter, FL 33408

SUBJECT: ST. LUCIE PLANT, UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENTS REGARDING TECHNICAL SPECIFICATION CHANGES RELATED TO COMPONENT CYCLIC OR TRANSIENT LIMITS (CAC NOS. MF8966 AND MF8967)

Dear Mr. Nazar:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment Nos. 241 and 192 to Renewed Facility Operating License Nos. DPR-67 and NPF-16 for the St. Lucie Plant, Unit Nos. 1 and 2 (St. Lucie 1 and 2), respectively. These amendments consist of changes to the Technical Specifications (TSs) in response to Florida Power & Light Company's application dated December 22, 2016.

The amendments update the St. Lucie 1 and 2 TSs to relocate the Component Cyclic or Transient Limits Program requirements to the Administrative Controls sections of the TSs. The amendments also delete the Component Cyclic or Transient Limits TS tables, which detail the allowable transient limits, and place these tables in licensee-controlled documents.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Perry H. Buckberg, Senior Project Manager
Plant Licensing Branch II-2
Division of Operator Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-335 and 50-389

Enclosures:
1. Amendment No. 241 to DPR-67
2. Amendment No. 192 to NPF-16
3. Safety Evaluation

cc w/enclosures: Distribution via Listserv

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 241
Renewed License No. DPR-67

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power & Light Company (FPL, the licensee), dated December 22, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Renewed Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 3.B to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 241, are hereby incorporated in the renewed license. FPL shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Farideh E Saba

for Undine S. Shoop, Chief
Plant Licensing Branch II-2
Division of Operator Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and
Technical Specifications

Date of Issuance: October 5, 2017

ATTACHMENT TO LICENSE AMENDMENT NO. 241

ST. LUCIE PLANT, UNIT NO. 1

RENEWED FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace page 3 of Renewed Facility Operating License No. DPR-67 with the attached revised page 3.

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Page	Insert Page
XIII	XIII
5-6a	5-6a
5-7	5-7
6-15h	6-15h

applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

A. Maximum Power Level

FPL is authorized to operate the facility at steady state reactor core power levels not in excess of 3020 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 241, are hereby incorporated in the renewed license. FPL shall operate the facility in accordance with the Technical Specifications.

Appendix B, the Environmental Protection Plan (Non-Radiological), contains environmental conditions of the renewed license. If significant detrimental effects or evidence of irreversible damage are detected by the monitoring programs required by Appendix B of this license, FPL will provide the Commission with an analysis of the problem and plan of action to be taken subject to Commission approval to eliminate or significantly reduce the detrimental effects or damage.

C. Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on March 28, 2003, describes certain future activities to be completed before the period of extended operation. FPL shall complete these activities no later than March 1, 2016, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on March 28, 2003, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed license. Until that update is complete, FPL may make changes to the programs described in such supplement without prior Commission approval, provided that FPL evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

D. Sustained Core Uncovery Actions

Procedural guidance shall be in place to instruct operators to implement actions that are designed to mitigate a small-break loss-of-coolant accident prior to a calculated time of sustained core uncovery.

INDEX

DESIGN FEATURES

DRAINAGE

5.6.2 The fuel pool is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 56 feet.

CAPACITY

5.6.3 The spent fuel pool storage racks are designed and shall be maintained with a storage capacity limited to no more than 1706 fuel assemblies, and the cask pit storage rack is designed and shall be maintained with a storage capacity limited to no more than 143 fuel assemblies. The total Unit 1 spent fuel pool and cask pit storage capacity is limited to no more than 1849 fuel assemblies.

5.7 SEISMIC CLASSIFICATION

5.7.1 Those structures, systems and components identified as seismic Class I in Section 3.2.1 of the FSAR shall be designed and maintained to the original design provisions contained in Section 3.7 of the FSAR with allowance for normal degradation pursuant to the applicable Surveillance Requirement.

5.8 METEOROLOGICAL TOWER LOCATION

5.8.1 The meteorological tower location shall be as shown on Figure 5.1-1.

5.9 DELETED

DELETED

o. Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies. The program shall ensure that Surveillance Requirements specified in the Technical Specifications are performed at intervals sufficient to assure the associated Limiting Conditions for Operation are met.

a. The Surveillance Frequency Control Program shall contain a list of frequencies of those Surveillance Requirements for which the frequency is controlled by the program.

b. Changes to the frequencies listed in the Surveillance Frequency Control Program shall be made in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1.

c. The provisions of Surveillance Requirements 4.0.2 and 4.0.3 are applicable to the frequencies established in the Surveillance Frequency Control Program.

p. Snubber Testing Program

This program conforms to the examination, testing and service life monitoring for dynamic restraints (snubbers) in accordance with 10 CFR 50.55a inservice inspection (ISI) requirements for supports. The program shall be in accordance with the following:

1. This program shall meet 10 CFR 50.55a(g) ISI requirements for supports.

2. The program shall meet the requirements for ISI of supports set forth in subsequent editions of the Code of Record and addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure (BPV) Code and the ASME Code for Operation and Maintenance of Nuclear Power Plants (OM Code) that are incorporated by reference in 10 CFR 50.55a(b) subject to the conditions listed in 10 CFR 50.55a(b) and subject to Commission approval.

3. The program shall, as required by 10 CFR 50.55a(b)(3)(v), meet Subsection ISTA, "General Requirements" and Subsection ISTD, "Preservice and Inservice Examination and Testing of Dynamic Restraints (Snubbers) in Light-Water Reactor Nuclear Power Plants".

4. The 120-month program updates shall be made in accordance with 10 CFR 50.55a(g)(4), 10 CFR 50.55a(g)(3)(v) and 10 CFR 50.55a(b) (including 10 CFR 50.55a(b)(3)(v)) subject to the conditions listed therein.

q. Component Cyclic or Transient Limit Program

The program provides controls to track the FSAR, Section 5.2, cyclic and transient occurrences to ensure that components are maintained within the design limits.

UNITED STATES
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WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

ORLANDO UTILITIES COMMISSION OF THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 192
Renewed License No. NPF-16

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power & Light Company (FPL, the licensee), dated December 22, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Renewed Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 3.B to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 192, are hereby incorporated in the renewed license. FPL shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Farideh E. Saba

for Undine S. Shoop, Chief
Plant Licensing Branch II-2
Division of Operator Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and
Technical Specifications

Date of Issuance: October 5, 2017

ATTACHMENT TO LICENSE AMENDMENT NO. 192

ST. LUCIE PLANT, UNIT NO. 2

RENEWED FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace page 3 of Renewed Facility Operating License No. NPF-16 with the attached revised page 3.

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Page	Insert Page
XXV	XXV
5-4a	5-4a
5-5	5-5
5-6	5-6
5-7	5-7
5-8	5-8
6-16	6-16

neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required.

D. Pursuant to the Act and 10 CFR Parts 30, 40, and 70, FPL to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

E. Pursuant to the Act and 10 CFR Parts 30, 40, and 70, FPL to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission's regulations: 10 CFR Part 20, Section 30.34 of 10 FR Part 30, Section 40.41 of 10 CFR Part 40, Section 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

FPL is authorized to operate the facility at steady state reactor core power levels not in excess of 3020 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 192, are hereby incorporated in the renewed license. FPL shall operate the facility in accordance with the Technical Specifications.

LIST OF TABLES (Continued)

CRITICALITY (continued)

5.6.1 c. (continued)

4. The 2x2 array of fuel assemblies that span the interface between Region 1 and Region 2 of the spent fuel pool storage racks shall comply with the storage pattern definitions of Figure 5.6-3 and the minimum burnup requirements as defined in Table 5.6-1. The allowed special arrangements in Region 2 as shown in Figure 5.6-2 shall not be placed adjacent to Region 1. (See Specification 5.6.1.c.7 for exceptions)

5. Fuel placed in the cask pit storage rack shall comply with the storage pattern definitions of Figure 5.6-4 and the minimum burnup requirements as defined in Table 5.6-1. (See Specification 5.6.1.c.7 for exceptions)

6. The same directional orientation for Metamic inserts is required for contiguous groups of 2x2 arrays where Metamic inserts are required.

7. Fresh or spent fuel in any allowed configuration may be replaced with non-fuel hardware, and fresh fuel in any allowed configuration may be replaced with a fuel rod storage basket containing fuel rod(s). Also, storage of Metamic inserts or control rods, without any fissile material, is acceptable in locations designated as completely water-filled cells.

d. The new fuel storage racks are designed for dry storage of unirradiated fuel assemblies having a maximum planar average U-235 enrichment less than or equal to 4.6 weight percent, while maintaining a k_{eff} of less than or equal to 0.98 under the most reactive condition.

DRAINAGE

5.6.2 The spent fuel storage pool is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 56 feet.

CAPACITY

5.6.3 The spent fuel pool storage racks are designed and shall be maintained with a storage capacity limited to no more than 1491 fuel assemblies, and the cask pit storage rack is designed and shall be maintained with a storage capacity limited to no more than 225 fuel assemblies. The total Unit 2 spent fuel pool and cask pit storage capacity is limited to no more than 1716 fuel assemblies.

5.7 DELETED

DELETED

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DELETED

DELETED

q. Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies. The program shall ensure that Surveillance Requirements specified in the Technical Specifications are performed at intervals sufficient to assure the associated Limiting Conditions for Operation are met.

a. The Surveillance Frequency Control Program shall contain a list of frequencies of those Surveillance Requirements for which the frequency is controlled by the program.

b. Changes to the frequencies listed in the Surveillance Frequency Control Program shall be made in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1.

c. The provisions of Surveillance Requirements 4.0.2 and 4.0.3 are applicable to the frequencies established in the Surveillance Frequency Control Program.

r. Component Cyclic or Transient Limit Program

The Program provides controls to track the FSAR, Section 3.9, cyclic and transient occurrences to ensure that components are maintained within the design limits.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the NRC.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier; and (4) modifications that may have significantly altered the nuclear, thermal or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 241 AND 192

TO RENEWED FACILITY OPERATING LICENSE NOS. DPR-67 AND NPF-16

FLORIDA POWER AND LIGHT COMPANY, ET AL.,

ST. LUCIE PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-335 AND 50-389

1.0 INTRODUCTION

By application dated December 22, 2016 (Reference 1), Florida Power & Light Company (FPL, the licensee) submitted to the U.S. Nuclear Regulatory Commission (NRC, the Commission) a license amendment request (LAR) for the St. Lucie Plant, Unit Nos. 1 and 2 (St. Lucie 1 and 2).

The proposed amendments would revise the St. Lucie 1 and 2 Technical Specifications (TSs) to relocate the Component Cyclic or Transient Limits Program requirements of St. Lucie 1 TS 5.9.1 and St. Lucie 2 TS 5.7.1 from the Design Features to the Administrative Controls sections of the St. Lucie 1 and 2 TSs, respectively. In addition, the proposed amendments delete the Component Cyclic or Transient Limits Table 5.9-1 and Table 5.7-1 from the St. Lucie 1 and 2 TSs and place these tables in the St. Lucie 1 Updated Final Safety Analysis Report (UFSAR) and St. Lucie 2 UFSAR, respectively.

2.0 REGULATORY EVALUATION

2.1 Description of the St. Lucie 1 and 2 Component Cyclic or Transient Limits

The component cyclic or transient limits were established to ensure that the associated components are maintained within their design limits over the lifetime of the plants. The purpose of setting the limits is to ensure the cumulative fatigue usage factors of the associated components due primarily to temperature and pressure cycling are within the American Society of Mechanical Engineers (ASME) Code allowable limit of 1.0. The limits are used to prevent crack initiation of the associated components and are not for safety limits, limiting safety settings, limiting control settings, limiting conditions for operation, or surveillance requirements.

2.2 Requested Changes

The requested changes would relocate the St. Lucie 1 and 2 Component Cyclic or Transient Limits Program requirements from the Design Features section to the Administrative Controls section of the St. Lucie 1 and 2 TSs. The requested change would also delete the St. Lucie 1 and 2 TSs Component Cyclic or Transient Limits Table 5.9-1 and Table 5.7-1, respectively, and place these tables in the St. Lucie 1 and 2 UFSARS.

2.3 Regulatory Review

Title 10 of the *Code of Federal Regulations* (10 CFR) Sections 50.36(c)(1), (2), and (3), specify that the TSs shall include safety limits, limiting safety system settings, limiting control settings, limiting conditions for operation, and surveillance requirements. Section 50.36(c)(4) of 10 CFR specifies that the design features to be included in the TSs are those features of the facility such as material of construction and geometry arrangements, which, if altered or modified, would have a significant effect on safety and are not covered in the categories described in 10 CFR 50.36(c)(1), (2), and (3). The component cyclic or transient limits are listed in the respective St. Lucie 1 and 2 TSs as a design feature and, as stated in Section 2.1 of this safety evaluation, the component cyclic or transient limits are not for safety limits, limiting safety settings, limiting control settings, limiting conditions for operation, or surveillance requirements. The licensee stated that the component cyclic or transient limits are not facility features that would have a significant effect on safety if altered or modified. The NRC staff agrees with the licensee's statement and confirmed that the component cyclic or transient limits do not meet the requirements defined in 10 CFR 50.36(c)(4).

NUREG-1432, Revision 4.0, "Standard Technical Specifications, Combustion Engineering Plants" (Reference 2), contain the improved Standard Technical Specifications (STS) for Combustion Engineering plants. The changes reflected in Revision 4.0 result from (1) the experience gained from plant operation using the improved STS and (2) extensive public technical meetings and discussions among the NRC staff and various nuclear power plant licensees and the Nuclear Steam Supply System (NSSS) Owners Groups. The improved STS were developed based on the criteria in the Commission's Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors, which was published in the *Federal Register* on July 22, 1993 (58 FR 39132), and was subsequently codified by changes to 10 CFR 50.36. Licensees are encouraged to upgrade their TSs consistent with those criteria, and conforming, to the extent practical, to Revision 4 of the improved STS. The Commission continues to place the highest priority on requests for complete conversions to the improved STS. Since St. Lucie 1 and 2 are Combustion Engineering plants, the licensee's proposed relocation of the Component Cyclic or Transient Limit Program to the Administrative Controls section is conforming to Section 5.5.5 of NUREG-1432, Revision 4.0.

Based on this regulatory review, the component cyclic or transient limits do not meet the 10 CFR 50.36(c) criteria for inclusion in the TSs. Therefore, the licensee's relocation of the component cyclic or transient limits from the Design Features section of the St. Lucie 1 and 2 TSs to the Administrative Controls section is acceptable.

3.0 TECHNICAL EVALUATION

The licensee states that the St. Lucie 1 and 2 TS Component Cyclic or Transient Limit Programs will be relocated to the Administrative Controls section of the St. Lucie 1 and 2 TSs and will provide controls to track the UFSAR, Section 5.2 (Unit 1)/Section 3.9 (Unit 2), cyclic and transient occurrences to ensure that components are maintained within the design limits. The licensee also proposes to delete Table 5.9-1 and Table 5.7-1 from the St. Lucie 1 and 2 TSs, respectively, and place these tables in St. Lucie 1 and 2 UFSAR Sections 5.2 and 3.9. The NRC staff reviewed the St. Lucie 1 and 2 UFSARs and confirmed that Unit 1, Section 5.2.1.2, "Transients Used in Design and Fatigue Analyses," and Unit 2, Section 3.9.1.1, "Design Transients," include detailed lists of the design transients. The content of these UFSAR sections are compatible with the Unit 1 TS, Table 5.9-1 and Unit 2 TS, Table 5.7-1. Therefore,

the NRC staff finds that moving these tables from the TSs to the UFSARs as described is acceptable.

In addition, the licensee described a fatigue monitoring program as the confirmatory program for fatigue of ASME Class 1 components in the reactor coolant system to assure that the design cycle and transient limits are not exceeded. The NRC staff's review and acceptance of the fatigue monitoring program is documented in NUREG-1779, "Safety Evaluation Report Related to the License Renewal of St. Lucie Nuclear Plant, Units 1 and 2," (Reference 3).

3.1 Technical Conclusion

The NRC staff reviewed the LAR for the Component Cyclic or Transient Limit Program and concluded that the licensee provided reasonable assurance that the reactor coolant system components will remain within their design limits over the design life to perform their intended design function. On the basis of the above evaluation, the staff concludes that the proposed LAR relocation of the Component Cyclic or Transient Program is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, on August 15, 2017, the NRC staff notified the State of Florida official (Mr. Clark Eldredge, Environmental Administrator, Bureau of Radiation Control, Florida Department of Health) of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on this finding published in the *Federal Register* on February 28, 2017 (82 FR 12133). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

7.0 REFERENCES

1 Costanzo, Christopher, Florida Power & Light Company, letter to U.S. Nuclear Regulatory Commission, "St. Lucie Units 1 and 2, Docket Nos. 50-335 and 50-389, Renewed Facility Operating License Nos. DPR-67 and NPF-16, License Amendment Request to Relocate the Component Cyclic and Transient Limits from the Technical Specifications to Licensee-Controlled Documents," December 22, 2016 (ADAMS Accession No. ML17006A007).

2 U.S. Nuclear Regulatory Commission, NUREG-1432, Revision 4.0, "Standard Technical Specifications, Combustion Engineering Plants" (ADAMS Accession Nos. ML12102A165 and ML12102A169).

3 U.S. Nuclear Regulatory Commission, NUREG-1779, "Safety Evaluation Report Related to the License Renewal of St. Lucie Nuclear Plant, Units 1 and 2," (ADAMS Package Accession No. ML031890043).

Principal Contributors: Robert Hsu
Perry Buckberg

Date: October 5, 2017

SUBJECT: ST. LUCIE PLANT, UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENTS REGARDING TECHNICAL SPECIFICATION CHANGES RELATED TO COMPONENT CYCLIC OR TRANSIENT LIMITS (CAC NOS. MF8966 AND MF8967) DATED OCTOBER 5, 2017

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