



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 151
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, (the licensee), dated December 20, 1996, as supplemented February 13, and April 17, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 151, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

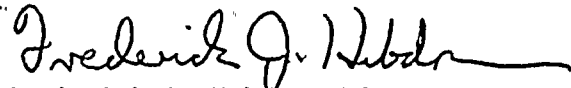
Also, the license is amended by adding a new License Condition, Paragraph 2.C.(6), to page 3a of the Facility Operating License DPR-67, as follows:

(6) Sustained Core Uncovery Actions

Procedural guidance shall be in place to instruct operators to implement actions which are designed to mitigate a small break loss of coolant accident prior to a calculated time of sustained core uncovery (Amendment No. 151).

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments: 1. Page 3a of license*
2. Changes to the Technical Specifications

Date of Issuance: May 16, 1997

*Page 3a is attached, for convenience, for the composite license to reflect this change.

DELETED PER
AMDT 89
1-13-88

~~(4) Prior to reaching 38,000 MWd/KTU peak assembly, the licensee must use an approved method to show that Combustion Engineering fuel will not experience creep collapse unless the new Exxon Corporation methodology has been approved for use by the staff and its results are valid for Cycle 6.~~

~~Added
per
Amdt. # 3
3-1-84~~

~~(5) The licensee shall provide a supplement to XN NF 85-117, "St. Lucie Unit 1 Revised LOCA ECCS Analysis with 15% Steam Generator Tube Plugging", that will provide the complete large break LOCA spectrum results to demonstrate full compliance with the criteria of 10 CFR 50.46 and Appendix K to 10 CFR Part 50 for the Commission staff's review and approval.~~

~~Amdt =
70
1-13-88~~

(6) Sustained Core Uncovery Actions

Procedural guidance shall be in place to instruct operators to implement actions which are designed to mitigate a small break loss of coolant accident prior to a calculated time of sustained core uncovery (Amendment No. 151).

2.D The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "St. Lucie Plant Security Plan," with revisions submitted through April 11, 1988; "St. Lucie Plant Training and Qualification Plan," with revisions submitted through August 8, 1985; and "St. Lucie Plant Safeguards Contingency Plan," with revisions submitted through December 8, 1986. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

~~Amdt.
97,
7-5-88~~

ATTACHMENT TO LICENSE AMENDMENT NO. 151

TO FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

Remove Page

2-1

Insert Page

2-1

2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

2.1 SAFETY LIMITS

REACTOR CORE

2.1.1 The combination of THERMAL POWER, pressurizer pressure, and maximum cold leg coolant temperature shall not exceed the limits shown on Figure 2.1-1.

APPLICABILITY: MODES 1 and 2.

ACTION:

Whenever the point defined by the combination of maximum cold leg temperature and THERMAL POWER has exceeded the appropriate pressurizer pressure line, be in HOT STANDBY within 1 hour.

REACTOR COOLANT SYSTEM PRESSURE

2.1.2 The Reactor Coolant System pressure shall not exceed 2750 psia.

APPLICABILITY: MODES 1, 2, 3, 4 and 5.

ACTION:

MODES 1 and 2

Whenever the Reactor Coolant System pressure has exceeded 2750 psia, be in HOT STANDBY with the Reactor Coolant System pressure within its limit within 1 hour.

MODES 3, 4 and 5

Whenever the Reactor Coolant System pressure has exceeded 2750 psia, reduce the Reactor Coolant System pressure to within its limit within 5 minutes.

Amendment No. ~~445~~
151

Correction letter of 7-12-96