

St. Lucie Unit 1 and Unit 2
Docket Nos. 50-335 and 50-389
Proposed License Amendments
Onsite Facility Review Group

ATTACHMENT 3

ST. LUCIE UNIT 1 MARKED-UP TECHNICAL SPECIFICATION PAGES

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INSERT - A, B

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6.0 ADMINISTRATIVE CONTROLS

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the FRG Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in FRG activities at any one time.

MEETING FREQUENCY

6.5.1.4 The FRG shall meet at least once per calendar month and as convened by the FRG Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the FRG necessary for the performance of the FRG responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Facility Review Group shall be responsible for:

- insert* *new* *add* *all procedure*
- a. Review of (1) *insert* all *new* procedures required by Specification 6.8 and *add* changes thereto; (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Plant General Manager to affect nuclear safety. *that require a written safety evaluation.*
 - b. Review of all proposed tests and experiments that affect nuclear safety.
 - c. Review of all proposed changes to Appendix A Technical Specifications.
 - d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
 - e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the President-Nuclear Division and to the Chairman of the Company Nuclear Review Board.
 - f. Review of all REPORTABLE EVENTS.
 - g. Review of unit operations to detect potential nuclear safety hazards.
 - h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant General Manager or the Company Nuclear Review Board.

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- i. Not Used.
- j. Not Used.
- k. Review of every unplanned on-site release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the President-Nuclear Division and to the Company Nuclear Review Board.
- l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL and RADWASTE TREATMENT SYSTEMS.
- m. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last FRG meeting.
- n. Review of the Fire Protection Program and implementing procedures and submittal of recommended changes to the Company Nuclear Review Board.

AUTHORITY

6.5.1.7

The Facility Review ^{Group} Board shall:

- a. ~~Recommend in writing to the Plant General Manager~~ approval or disapproval of items considered under Specifications 6.5.1.6.a through d above, ^{as certified} in writing by the FRG Chairman or his designated alternate,
- b. Render determinations in writing with regard to whether or not each item considered under Specifications 6.5.1.6.a, b, d, and e above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the President-Nuclear Division and the Company Nuclear Review Board of disagreement between the FRG and the Plant General Manager; however, the Plant General Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.1.8

The Facility Review Group shall maintain written minutes of each FRG meeting that, at a minimum, document the results of all FRG activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the President-Nuclear Division and the Chairman of the Company Nuclear Review Board.

^{insert}
Plant General Manager



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- c. The Safety Limit Violation Report shall be submitted to the Commission, the CNRB, and the President-Nuclear Division within 14 days of the violation.
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978, and those required for implementing the requirements of NUREG 0737.
- b. Refueling operations.
- c. Surveillance and test activities of safety-related equipment.
- d. Not Used.
- e. Not Used.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Control Program for effluent monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974.
- j. Quality Control Program for environmental monitoring using the guidance in Regulatory Guide 4.1, Revision 1, April 1975.

REPLACE with INSERT-A

6.8.2 ~~Each procedure of Specification 6.8.1a through i. above, and changes thereto, shall be reviewed by the FRG and shall be approved by the Plant General Manager prior to implementation and shall be reviewed periodically as set forth in administrative procedures.~~

6.8.3 b. Temporary changes to procedures of Specification 6.8.1a through i. above may be made provided:

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- a(1) The intent of the original procedure is not altered.
- b(2) The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.

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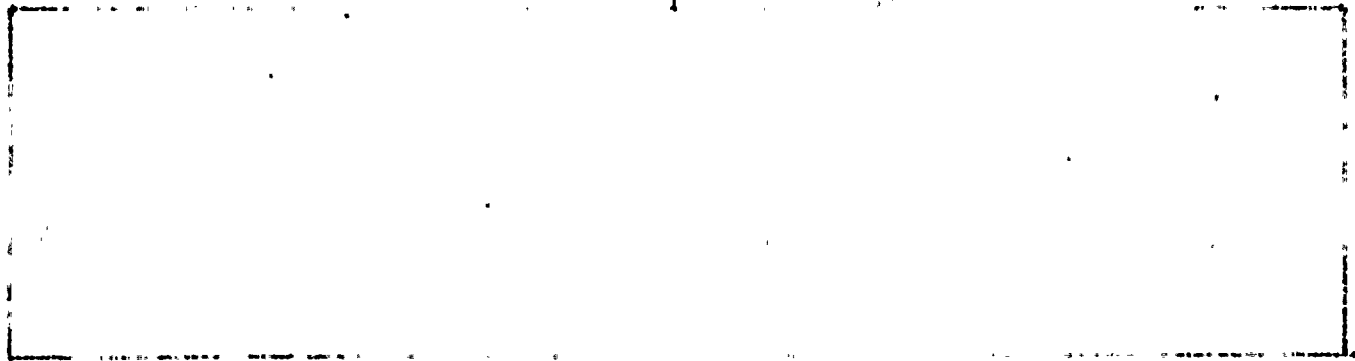
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- 6.8.2 Each new procedure of Specification 6.8.1.a through i. above, except 6.8.1.f, shall be reviewed and approved by the FRG prior to implementation. Fire Protection Program implementation procedures and changes thereto shall be reviewed by the FRG and approved by the Plant General Manager. Each procedure of Specification 6.8.1 shall be reviewed periodically as set forth in administrative procedures.

INSERT - B

6.8.3 Changes to Procedures

- a. Each revision to the procedures required by Specification 6.8.1.a through i. above shall be independently reviewed by an individual or group from the appropriate discipline(s), and except as required by Specifications 6.5.1.7 and 6.8.2, each revision shall be approved by the Plant General Manager or individual(s) designated in writing by the Plant General Manager prior to implementation. Personnel authorized to approve procedure changes pursuant to this Specification shall be at a management level appropriate for the activities affected by the procedure being revised.



6.0 ADMINISTRATIVE CONTROLS

and, if appropriate, incorporated in the next revision of the affected procedure pursuant

to Specification 6.8.3.a. REPLACE

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c. (3) The change is documented, reviewed by the ERG and approved by the Plant General Manager within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, maintained, and shall be audited under the cognizance of the CNRB:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the Shutdown Cooling System, High Pressure Safety Injection System, Containment Spray System, and RCS Sampling. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radioiodine Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for monitoring, and
- (iii) Provisions for maintenance of sampling and analysis equipment.

c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- (i) Identification of a sampling schedule for the critical variables and control points for these variables,
- (ii) Identification of the procedures used to measure the values of the critical variables,
- (iii) Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in-leakage,

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ATTACHMENT 4

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ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

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QUORUM

6.5.1.5 The quorum of the FRG necessary for the performance of the FRG responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Facility Review Group shall be responsible for:

- a. Review of (1) ^{new insert} all procedures required by Specification 6.8 and ^{insert} changes thereto, (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Plant General Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix A Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the President-Nuclear Division and to the Chairman of the Company Nuclear Review Board.
- f. Review of all REPORTABLE EVENTS.
- g. Review of unit operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant General Manager or the Company Nuclear Review Board.
- i. Not Used.
- j. Not Used.

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that require a written safety evaluation,

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ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- k. Review of every unplanned onsite release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the President - Nuclear Division and to the Company Nuclear Review Board.
- l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL and RADWASTE TREATMENT SYSTEMS.
- m. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last FRG meeting.
- n. Review of the Fire Protection Program and implementing procedures and submittal of recommended changes to the Company Nuclear Review Board.

AUTHORITY

6.5.1.7 The Facility Review Group shall:

- a. ~~Recommend in writing to the Plant General Manager approval or disapproval of items considered under Specifications 6.5.1.6a, through d. and m. above,~~ ^{Delete} *as certified in writing by the FRG Chairman or his designated alternate.* ^{Provide}
- b. Render determinations in writing with regard to whether or not each item considered under Specifications 6.5.1.6a, b, d and e above constitutes an unreviewed safety question. ^{Add}
- c. Provide written notification within 24 hours to the President-Nuclear Division and the Company Nuclear Review Board of disagreement between the FRG and the Plant General Manager; however, the Plant General Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.1.8 The Facility Review Group shall maintain written minutes of each FRG meeting that, at a minimum, document the results of all FRG activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the President - Nuclear Division and the Chairman of the Company Nuclear Review Board.

6.5.2 COMPANY NUCLEAR REVIEW BOARD (CNRB)

^{insert} Plant General Manager

FUNCTION

6.5.2.1 The Company Nuclear Review Board shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy



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ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Control Program for effluent monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974.
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REPLACE with INSERT-A

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6.8.3 b. Temporary changes to procedures of Specification 6.8.1a. through i. above may be made provided:

- a. (1) The intent of the original procedure is not altered.
- b. (2) The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected. *and, if appropriate, incorporated in the next revision of the affected procedure pursuant to Specification 6.8.3.a*
- a. (3) ~~The change is documented, reviewed by the FRG and approved by the Plant General Manager within 14 days of implementation.~~

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6.8.4 The following programs shall be established, implemented, maintained, and shall be audited under the cognizance of the CNRB:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the Shutdown Cooling System, High Pressure Safety Injection System, Containment Spray System, and RCS Sampling. The program shall include the following:

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A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

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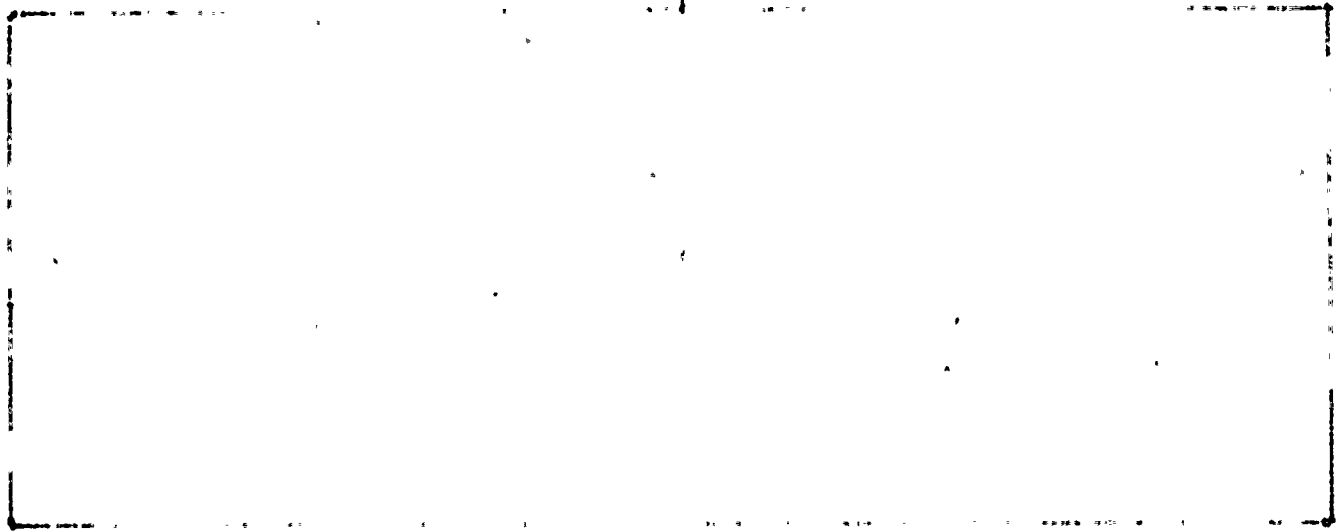
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