ATTACHMENT 3

PROPOSED LICENSE AMENDMENT FOR
IMPLEMENTATION OF 10 CFR 50, APPENDIX J OPTION B

PROPOSED TECHNICAL SPECIFICATIONS PAGES

1-2
3/4 6-1 thru 3/4 6-11
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B3/4 6-1
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6-15b

Insert Pages

CHANNEL FUNCTIONAL TEST

1.6 A CHANNEL FUNCTIONAL TEST shall be the injection of a simulated signal into the channel as close to the primary sensor as practicable to verify OPERABILITY including alarm and/or trip functions.

CONTAINMENT VESSEL INTEGRITY

1.7 CONTAINMENT VESSEL INTEGRITY shall exist when:

a. All containment vessel penetrations required to be closed during accident conditions are either:

1. Capable of being closed by an OPERABLE containment automatic isolation valve system, or

2. Closed by manual valves, blind flanges, or deactivated automatic valves secured in their closed position ~~except as provided in Table 3.6-2 of Specification 3.6.3.1.~~ for valves that are open on an intermittent basis under administrative Control.

b. All containment vessel equipment hatches are closed and sealed,

c. Each containment vessel air lock is in compliance with the requirements of Specification 3.6.1.3,

d. The containment leakage rates are within the limits of Specification 3.6.1.2, and

e. The sealing mechanism associated with each penetration (e.g., welds, bellows or O-rings) is OPERABLE.

CONTROLLED LEAKAGE

1.8 CONTROLLED LEAKAGE shall be the seal water flow supplied from the reactor coolant pump seals.

CORE ALTERATION

1.9 CORE ALTERATION shall be the movement or manipulation of any fuel, sources, reactivity control components, or other components affecting reactivity within the reactor vessel with the vessel head removed and fuel in the vessel. Exceptions to the above include shared (4 fingered) control element assemblies (CEAs) withdrawn into the upper guide structure (UGS) or evolutions performed with the UGS in place such as CEA latching/unlatching or verification of latching/unlatching which do not constitute a CORE ALTERATION. Suspension of CORE ALTERATIONS shall not preclude completion of movement of a component to a safe position.

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 CONTAINMENT VESSEL

CONTAINMENT VESSEL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 CONTAINMENT VESSEL INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

Without CONTAINMENT VESSEL INTEGRITY, restore CONTAINMENT VESSEL INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 CONTAINMENT VESSEL INTEGRITY shall be demonstrated:

a. At least once per 31 days by verifying that:

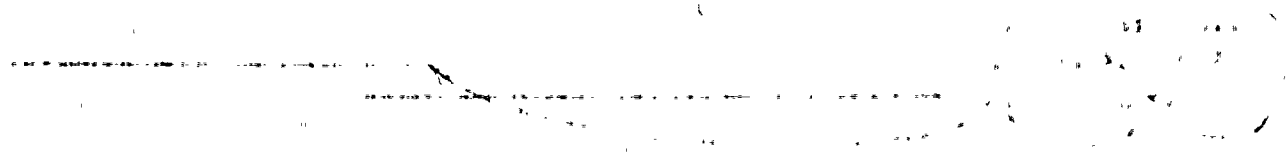
1. All containment vessel penetrations* not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except ~~as provided in Table 3.6-2 of Specification 3.6.3.1~~, and

for valves that are open on an intermittent basis under administrative control.

2. All containment vessel equipment hatches are closed and sealed.

b. By verifying that each containment vessel air lock is OPERABLE per Specification 3.6.1.3.

*Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked, sealed or otherwise secured in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days.



CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited ~~to:~~ in accordance with the Containment Leakage Rate Testing Program.

~~a. An overall integrated leakage rate of:~~

~~1. $\leq L_a$, 0.50 percent by weight of the containment air per 24 hours at P_a, (39.6 psig), or~~

~~2. $\leq L_t$, 0.32 percent by weight of the containment air per 24 hours at a reduced pressure of P_t, (19.8 psig).~~

~~b. A combined leakage rate of $\leq 0.60\ L_a$ for all penetrations and valves subject to Type B and C tests as identified in Table 3.6-1 when pressurized to P_a.~~

~~c. A combined leakage rate of $\leq 0.27\ L_a$ for all penetrations identified in Table 3.6-1 as secondary containment bypass leakage paths when pressurized to P_a.~~

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

[INSERT A]

~~With either (a) the measured overall integrated containment leakage rate exceeding $0.75\ L_a$ or $0.75\ L_t$, as applicable, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60\ L_a$, or (c) with the combined bypass leakage rate exceeding $0.27\ L_a$,~~ restore the overall leakage rate~~(s)~~ to ~~within the limit(s)~~ less than that specified by the Containment Leakage Rate Testing Program prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the required test Schedule ~~following test schedule~~ and shall be determined in conformance with the criteria specified in ~~Appendix J of 10 CFR 50:~~ the Containment Leakage Rate Testing Program.

~~a. Type A test shall be performed in accordance with 10 CFR 50 Appendix J, as modified by approved exemptions.~~

SURVEILLANCE REQUIREMENTS (Continued)

Pages 3/4 6-3 through 3/4 6-9 have been DELETED.
Page 3/4 6-10 is the next valid page.

b. If any periodic Type A test fails to meet either .75 L_a or .75 L_t, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet either .75 L_a or .75 L_t, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet either .75 L_a or .75 L_t at which time the above test schedule may be resumed.

c. The accuracy of each Type A test shall be verified by a supplemental test which:

1. Confirms the accuracy of the Type A test by verifying that the difference between supplemental and Type A test data is within 0.25 L_a or 0.25 L_t,

2. Has a duration sufficient to establish accurately the change in leakage between the Type A test and the supplemental test.

3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage rate at P_a (39.6 psig) or P_t (19.8 psig).

d. Type B and C tests shall be conducted with gas at P_a (39.6 psig) at intervals no greater than 24 months except for tests involving air locks.

e. The combined bypass leakage rate shall be determined to be $< 0.27\ L_a$ by applicable Type B and C tests at least once per 24 months except for penetrations which are not individually testable; penetrations not individually testable shall be determined to have no detectable leakage when tested with soap bubbles while the containment is pressurized to P_a (39.6 psig) during each Type A test.

f. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

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g. All Type A test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.

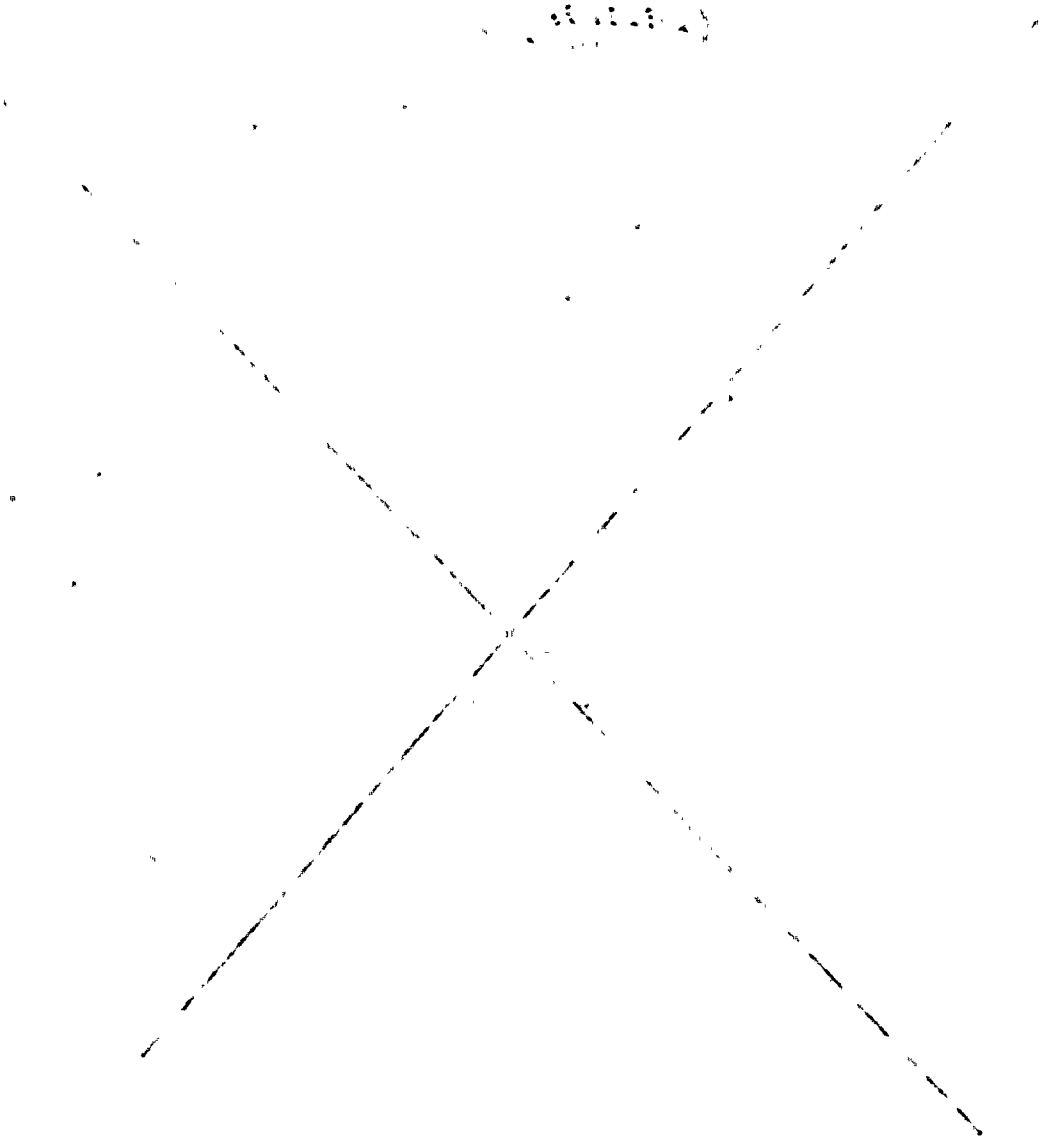


TABLE 3.6-1

CONTAINMENT LEAKAGE PATHS

Penetration	System	Valve Tag Number	Location to Containment	Service	Test Type*
7	Makeup Water	Gate (I-MV-15-1) Check (I-V-15328)	Outside Inside	Primary Makeup Water	Bypass
8	Station Air	Globe (I-V-18-794) Globe (I-V-18-796)	Outside Outside	Station Air Supply	Bypass
9	Instrument Air	Gate (I-MV-18-1) Check (I-V-18195)	Outside Inside	Instrument Air Supply	Bypass
10	Containment Purge	Butterfly (I-FCV-25-4) Butterfly (I-FCV-25-5)	Inside Outside	Containment Purge Exhaust	Type C
11	Containment Purge	Butterfly (I-FCV-25-3) Butterfly (I-FCV-25-2)	Inside Outside	Containment Purge Supply	Type C
14	Waste Management	Globe (V-6741) Check (V-6779)	Outside Outside	Nitrogen supply to SI Tanks	Bypass
23	Component Cooling	Butterfly (I-HCV 14-7) Butterfly (I-HCV-14-1)	Outside Outside	RC Pump CW Supply	Bypass
24	Component Cooling	Butterfly (I-HCV-14-6) Butterfly (I-HCV-14-2)	Outside Outside	RC Pump CW Return	Bypass
25	Fuel Transfer Tube	Double Gasket Flange	Inside	Fuel Transfer	Bypass
26	CVCS	Globe (V-2515) Globe (V-2516)	Inside Inside	Letdown Line	Bypass
28	Sampling	Globe (V-5200) Globe (V-5203) Globe (I-FCV-03-1E) Globe (I-FCV-03-1F)	Outside Outside Outside Outside	Reactor Coolant Sample SI Tank Sample SI Tank Sample	Bypass Bypass

TABLE 3.6-1 (Continued)

Penetration	System	Valve Tag Number	Location to Containment	Service	Test Type*
29	Sampling	Globe (V-5202) Globe (V-5205)	Outside Outside	Pressurizer Steam Space Sample	Bypass
29	Sampling	Globe (V-5201) Globe (V-5204)	Outside Outside	Pressurizer Surge Line Sample	Bypass
31	Waste Management	Gate (V-6554) Gate (V-6555)	Outside Outside	Containment Vent Header	Bypass
41	Safety Injection Tank Test Lines	Gate (V-3463) Gate (I-V-07009)	Outside Outside	Safety Injection Tank Fill and Sampling	Bypass
42	Waste Management	Gate (I-LCV-07-11A) Gate (I-LCV-07-11B)	Outside Outside	Reactor Cavity Sump Pump Discharge	Bypass
43	Waste Management	Gate (V-6301) Gate (V-6302)	Outside Outside	Reactor Drain Tank Pump Suction	Bypass
44	CVCS	Gate (V-2505) Gate (I-SE-01-1)	Outside Inside	RC Pump Controlled Bleedoff	Bypass
46	Fuel Pool Cleanup	Gate (I-V-07-206) Gate (I-V-07-189)	Outside Inside	Refueling Cavity Purification Flow Inlet	Bypass
47	Fuel Pool Cleanup	Gate (I-V-07-170) Gate (I-V-07-188)	Outside Inside	Refueling Cavity Purification Flow Outlet	Bypass
48a	Sampling	Globe (I-FSE-27-1, 2, 3, 4) Globe (I-FSE-27-8)	Inside Outside	H_2 Sampling	Type C

TABLE 3.6-1 (Continued)

Penetration	System	Valve Tag Number	Location to Containment	Service	Test Type*
48c	Sampling	Globe (I-FSE-27-11) Check (I-V-27101)	Outside Inside	H_2 Sampling	Type C
51c	Sampling	Globe (I-FSE-27-5,6,7) Globe (I-FSE-27-9)	Inside Outside	H_2 Sampling	Type C
51a	Sampling	Globe (I-FSE-27-10) Check (I-V-27102)	Outside Inside	H_2 Sampling	Type C
52a	Sampling	Gate (I-FCV-26-1) Gate (I-FCV-26-2)	Inside Outside	Radiation Monitoring	Bypass
52b	Sampling	Gate (I-FCV-26-3) Gate (I-FCV-26-4)	Inside Outside	Radiation Monitoring	Bypass
52c	Sampling	Gate (I-FCV-26-5) Gate (I-FCV-26-6)	Inside Outside	Radiation Monitoring Return	Bypass
52d	ILRT	Globe (I-V00140) Globe (I-V00143)	Inside Outside	ILRT Test Tap	Bypass
52e	ILRT	Globe (I-V00139) Globe (I-V00144)	Inside Outside	ILRT Test Tap	Bypass
54	ILRT	Blind Flange Gate (I-V00101)	Inside Outside	ILRT Pressure Connection	Bypass
56	Containment H_2 Purge	Gate (I-V-25-11) Gate (I-V-25-12)	Outside Outside	Hydrogen Purge Outside Air Makeup	Bypass
57	Containment H_2 Purge	Gate (I-V-25-13) Gate (I-V-25-14)	Outside Outside	Hydrogen Purge Exhaust	Bypass

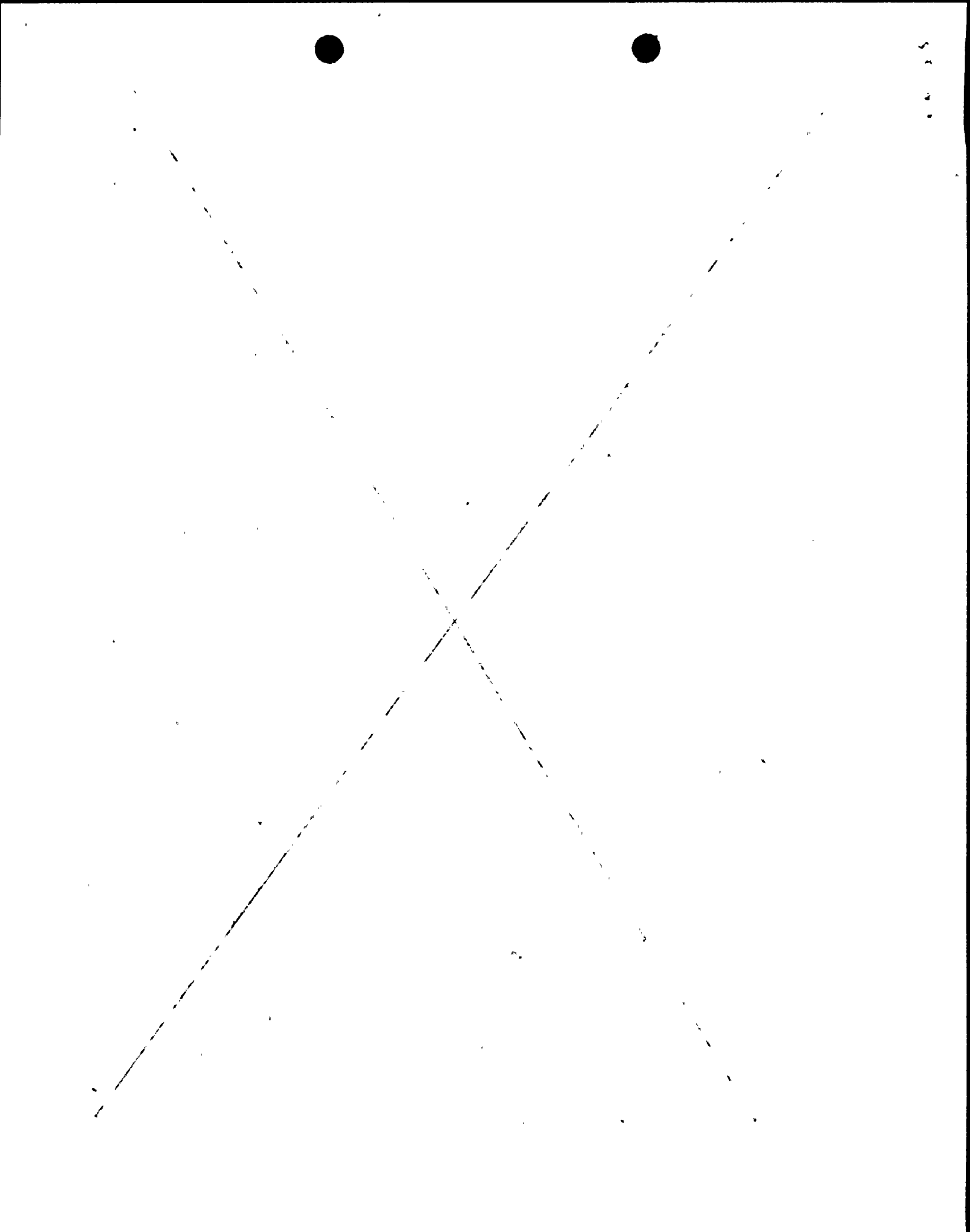


TABLE 3.6-1 (Continued)

Penetration	System	Valve Tag Number	Location to Containment	Service	Test Type*
58	Containment H_2 Purge	Gate (I-V-25-15) Gate (I-V-25-16)	Outside Outside	Hydrogen Purge Exhaust	Bypass
67	Vacuum Relief	Check (I-V-25-20) Butterfly (I-FCV-25-7)	Inside Outside	Containment Vacuum Relief	Type C
68	Vacuum Relief	Check (I-V-25-21) Butterfly (I-FCV-25-8)	Inside Outside	Containment Vacuum Relief	Type C
Personnel Lock	N.A.	None	N.A.	Ingress & Egress to Containment	Type B**
Escape Lock	N.A.	None	N.A.	Emergency Ingress & Egress to Containment	Type B**
Maintenance Hatch	N.A.	None	N.A.	Vessel Maintenance	Type B (Gasket Interspace)
Electrical Penetrations	N.A.	All primary canisters and flanged electrical penetrations except welded spares	N.A.	Electrical connections in PCV	Type B
1	Main Steam Steel Containment Nozzles	Tap 1 Tap 2	Outside Outside	Expansion Bellows	Type B
2	Main Steam Steel Containment Nozzles	Tap 1 Tap 2	Outside Outside	Expansion Bellows	Type B
3	Feedwater Steel Containment Nozzles	Tap 1 Tap 2	Outside Outside	Expansion Bellows	Type B

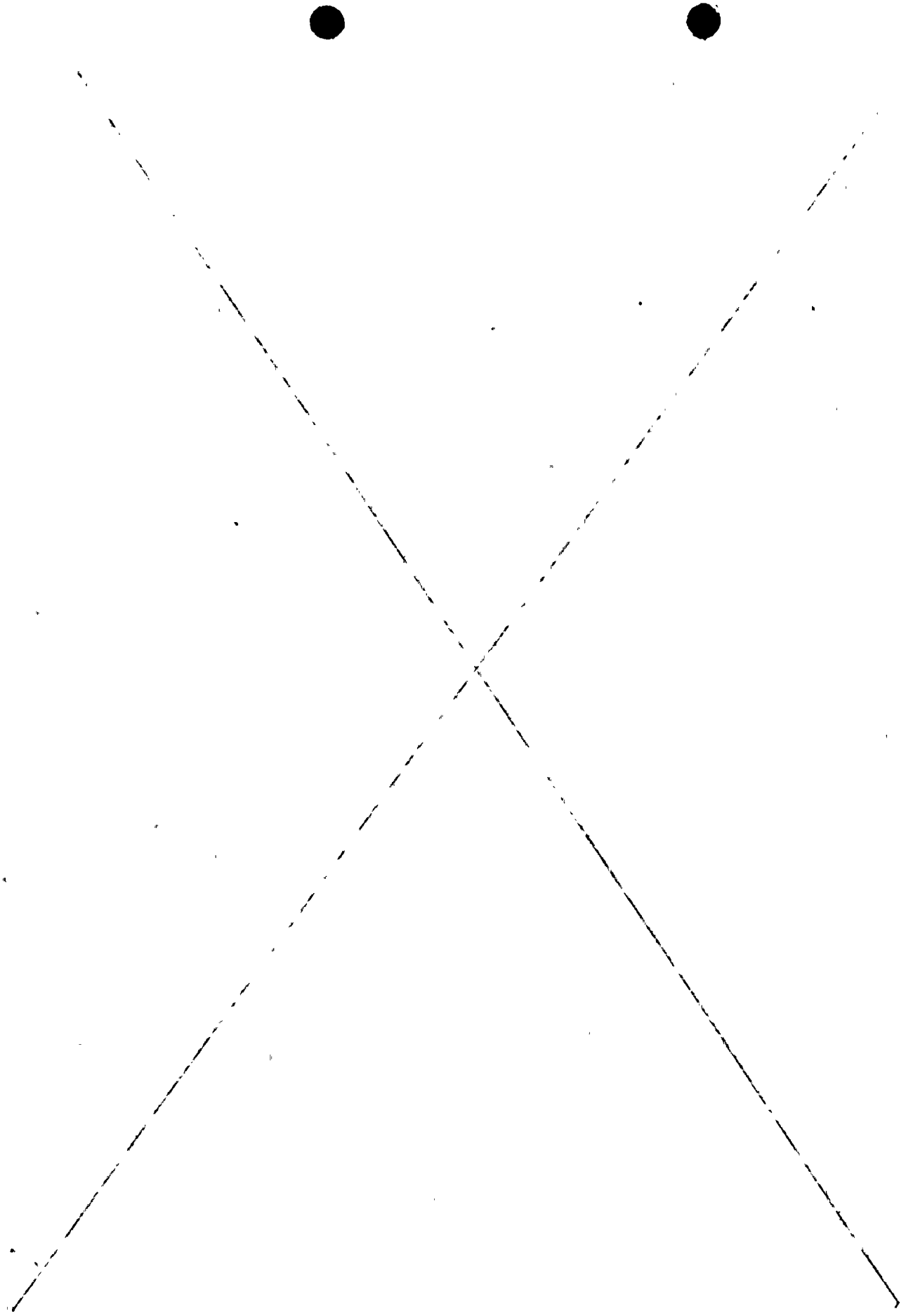
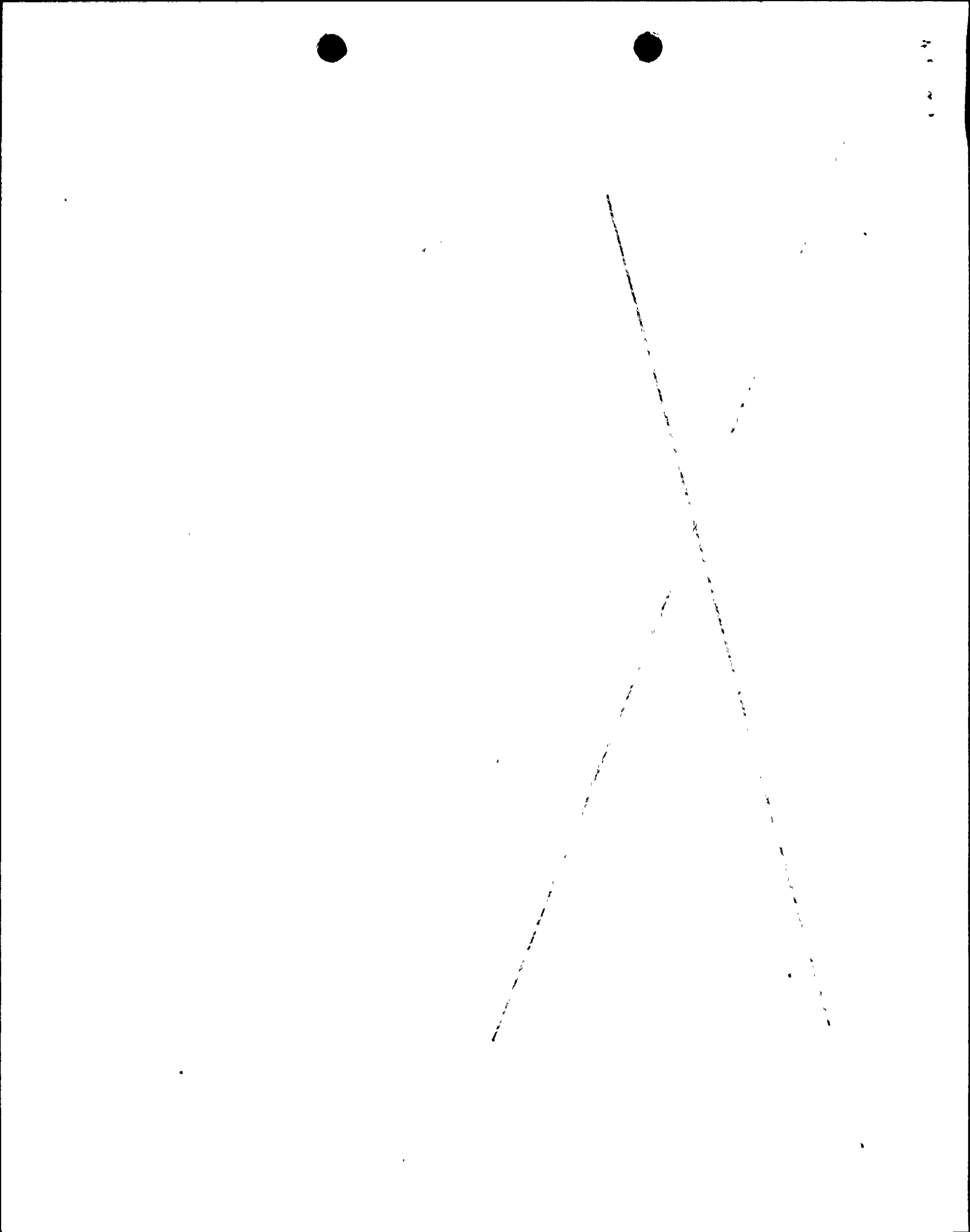


TABLE 3.6-1 (Continued)

Penetration	System	Valve Tag Number	Location to Containment	Service	Test Type*
4	Feedwater Steel Containment Nozzles	Tap 1 Tap 2	Outside Outside	Expansion Bellows	Type B
25	Fuel Tube Steel Containment Nozzles	Tap 1	Inside	Expansion Bellows	Type B

* Type C and bypass tests are conducted in the same manner, the only difference is in the acceptance criteria that is applicable.
** In accordance with Specification 4.6.1.3.b.



CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be OPERABLE with:

a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and

b. An overall air lock leakage rate ~~of less than or equal to 0.05 L_a at P_a, 39.6 psig.~~ in accordance with the Containment Leakage Rate Testing Program.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With one containment air lock door inoperable*:

1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.

2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be closed at least once per 31 days.

3. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

4. The provisions of Specification 3.0.4 are not applicable.

b. With the containment air lock inoperable, except as the result of an inoperable air lock door, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

~~a. Within 72 hours following each closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying the seal leakage is < 0.01 L_a as determined by precision flow measurement when the volume between the door seals is pressurized to greater than or equal to:~~

*If the inner air lock door is inoperable, passage through the OPERABLE outer air lock door is permitted to effect repairs to the inoperable inner air lock door. No more than one airlock door shall be open at any time.

SURVEILLANCE REQUIREMENTS (continued)

~~1. For the personnel air lock, greater than or equal to P_a, 39.6 psig for at least 15 minutes.~~

~~2. For the emergency air lock, greater than or equal to 10.0 psig for at least 15 minutes.~~

~~b. By conducting overall air lock leakage tests at not less than P_a, 39.6 psig, and verifying the overall air lock leakage rate is within its limit:~~

~~1. At least once per 6 months,# and~~

~~2. Prior to establishing CONTAINMENT INTEGRITY when maintenance has been performed on the air lock that could affect the air lock sealing capability.*~~

~~c.~~ b. At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

a. By verifying leakage rates and air lock door seals in accordance with the Containment Leakage Rate Testing Program; and

~~# The provisions of Specification 4.0.2 are not applicable.~~

~~* This constitutes an exemption to Appendix J of 10 CFR 50.~~

CONTAINMENT SYSTEMS

CONTAINMENT VESSEL STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.6 The structural integrity of the containment vessel shall be maintained at a level consistent with the acceptance criteria in Specification 4.6.1.6.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the structural integrity of the containment vessel not conforming to the above requirements, restore the structural integrity to within the limits prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.6 The structural integrity of the containment vessel shall be determined ~~during the shutdown for each Type A containment leakage rate test (reference Specification 4.6.1.2)~~ by a visual inspection of the accessible interior and exterior surfaces of the vessel and verifying no apparent changes in appearance of the surfaces or other abnormal degradation.

, in accordance with the Containment Leakage Rate Testing Program,

CONTAINMENT SYSTEMS

3/4.6.3 CONTAINMENT ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.6.3.1 The containment isolation valves ~~specified in Table 3.6-2~~ shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more of the isolation valve(s) ~~specified in Table 3.6-2~~ inoperable, either:

a. Restore the inoperable valve(s) to OPERABLE status within 4 hours, or

b. Isolate each affected penetration within 4 hours by use of at least one deactivated automatic valve secured in the isolation position, or

c. Isolate each affected penetration within 4 hours by use of at least one closed manual valve or blind flange; or

d. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.3.1.1 The isolation valves ~~specified in Table 3.6-2~~ shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair or replacement work is performed on the valve or its associated actuator, control or power circuit by performance of the cycling test, and verification of isolation time.

SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.1.2 Each Containment isolation valve ~~specified in Table 3.6-2~~ shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

a. Verifying that on a Containment Isolation test signal, and/or SIAS test signal, each isolation valve actuates to its isolation position.

4.6.3.1.3 The isolation time of each power operated or automatic Containment isolation valve ~~of Table 3.6-2~~ shall be determined to be within its limits when tested pursuant to Specification 4.0.5.

Pages 3/4 6-20 through 3/4 6-22 have been deleted.

TABLE 3.6-2

CONTAINMENT ISOLATION VALVES

Valve Tag Number	Penetration Number	Function	Testable During Plant Operation	Isolation Time (Sec)
A. CONTAINMENT ISOLATION				
1. I-FCV-25-4,5	10	Containment purge air exhaust, CIS	No	5
2. I-FCV-25-2,3	11	Containment purge supply, CIS	No	5
3. I-MV-15-1	7	Primary makeup water, CIS	Yes	19
4. I-MV-18-1	9	Instrument air supply, CIS	No	28
5. V-6741	14	Nitrogen supply to safety injection tanks, CIS	Yes	5
6. I-HCV-14-1 & 7	23	Reactor coolant pump cooling water supply, SIAS	No	5
7. I-HCV-14-6 & 2	24	Reactor coolant pump cooling water return, SIAS	No	5
8. V-2515,2516	26	Letdown line, CIS, SIAS	No	5
9. V-5200,5203	28	Reactor coolant sample, CIS	Yes	5
10. V-5201,5204	29	Pressurizer surge line sample, CIS	Yes	5
11. V-5202,5205	29	Pressurizer steam space sample, CIS	Yes	5
12. V-6554,6555	31	Containment vent header, CIS	Yes	5
13. I-LCV-07-11A,11B	42	Reactor cavity sump pump discharge, CIS	Yes	10
14. V-6301,6302	43	Reactor drain tank pump suction, CIS	Yes	5
15. V-2505	44	Reactor coolant pump controlled bleedoff, CIS	No	5
16. I-SE-01-1	44	Reactor coolant pump controlled bleedoff, CIS	No	5

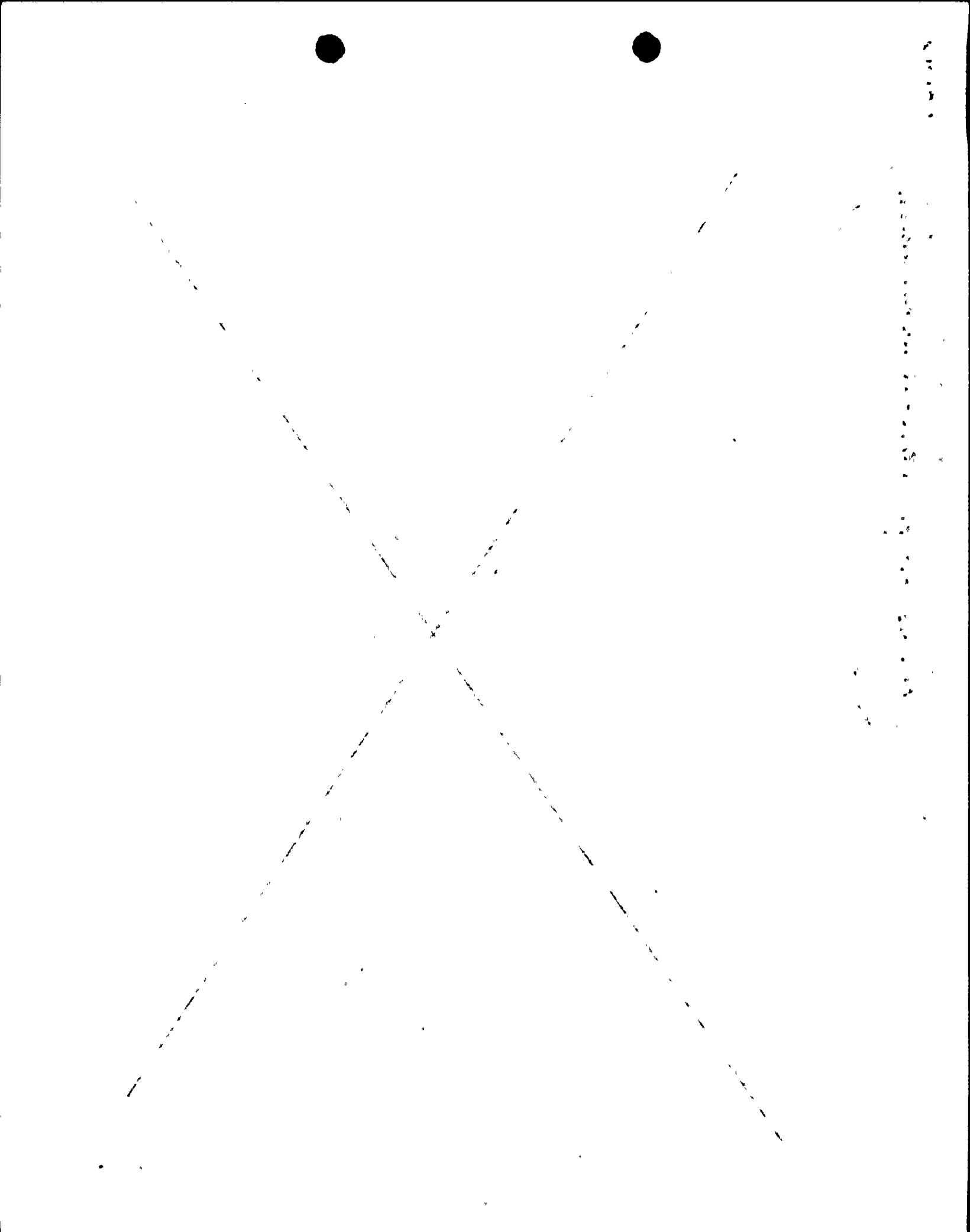


TABLE 3.6-2 (Continued)

Valve Tag Number	Penetration Number	Function	Testable During Plant Operation	Isolation Time (Sec)
B. MANUAL OR REMOTE MANUAL				
1. I-V-18-794 I-V-18-796	8	Station air supply, Manual	Yes	NA
2. I-V-25-11,12	56	Hydrogen purge outside air make-up, Manual (NC)	Yes	NA
3. I-V-25-13,14, 15,16	57 & 58	Hydrogen purge exhaust, Manual (NC)	Yes	NA
4. V-3463	41	Safety injection tank test line, Manual (NC)	Yes	NA*
5. I-V-07009	41	Safety injection tank test line, Manual (NC)	Yes	NA*
6. V-07206, V-07189	46	Refueling cavity purification flow inlet, Manual (NC)	Yes	NA
7. V-07170, V-07188	47	Refueling cavity purification flow outlet, Manual (NC)	Yes	NA
8. I-FSE-27-1,2,3, 4,8,11	48a & 48c	Hydrogen sampling line, Remote manual	Yes	NA*
9. I-FSE-27-5,6,7, 9,10	51a & 51c	Hydrogen sampling line, Remote manual	Yes	NA*

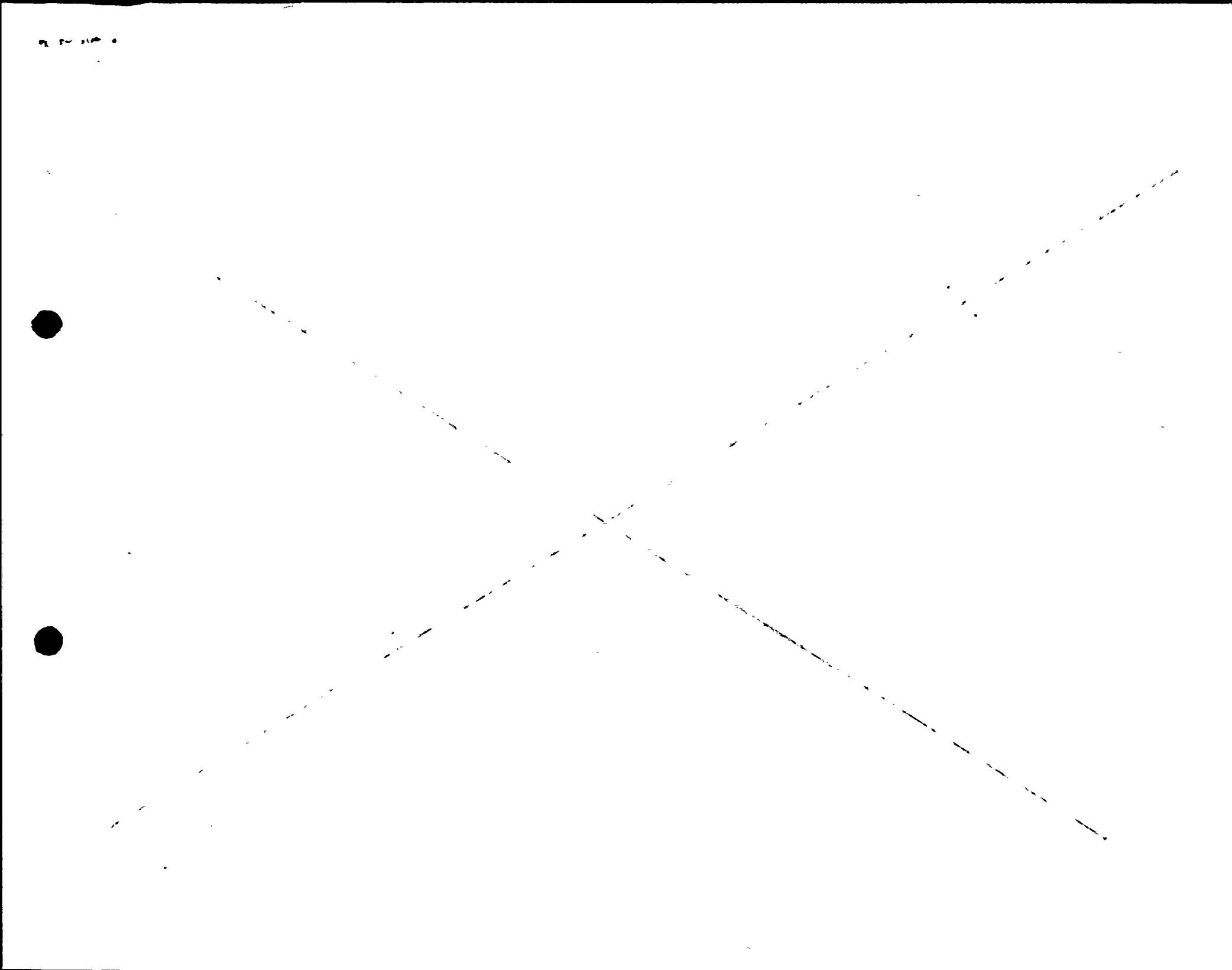
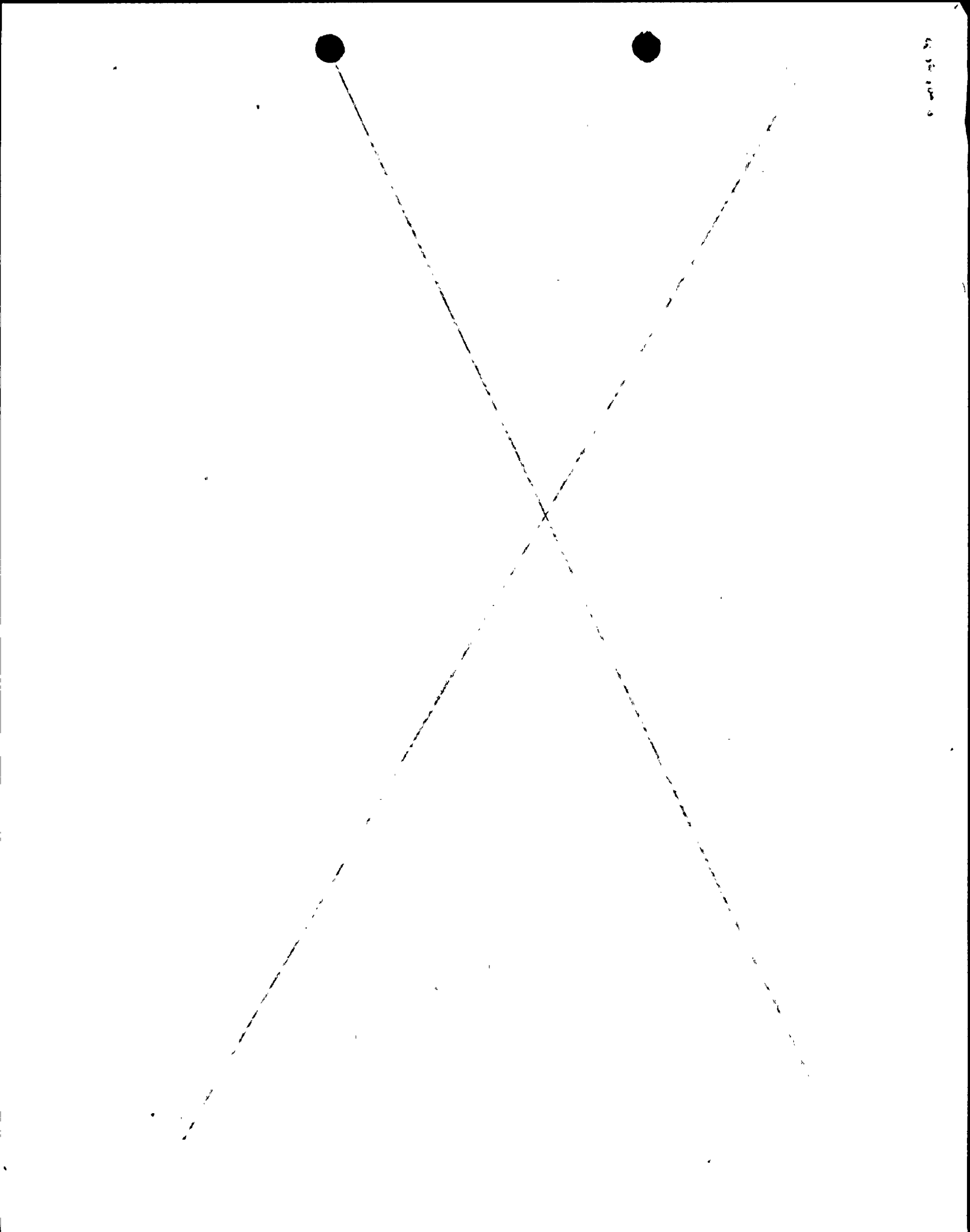


TABLE 3.6-2 (Continued)

Valve Tag Number	Penetration Number	Function	Testable During Plant Operation	Isolation Time (Sec)
10. I-FCV-26-1 & 2	52a	Radiation monitoring	Yes	NA
11. I-FCV-26-3 & 4	52b	Radiation monitoring	Yes	NA
12. I-FCV-26-5 & 6	52c	Radiation monitoring, return	Yes	NA
13. I-V00140 I-V00143	52d	ILRT test tap	Yes	NA
14. I-V00139 I-V00144	52e	ILRT test tap	Yes	NA
15. I-V00101	54	ILRT pressure connection	Yes	NA
16. I-FCV-03-1E & 1F	28	SI Tank Sample	Yes	NA**

NA - Manual Valve-Isolation time not applicable.
* May be opened on an intermittent basis under administrative control.
** Normally closed valves - Isolation time not applicable.



CONTAINMENT SYSTEMS

SHIELD BUILDING STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.6.3 The structural integrity of the shield building shall be maintained at a level consistent with the acceptance criteria in Specification 4.6.6.3.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION

With the structural integrity of the shield building not conforming to the above requirements, restore the structural integrity to within the limits prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.6.3 The structural integrity of the shield building shall be determined ~~during the shutdown for each Type A containment leakage rate test (reference Specification 4.6.1.2)~~ by a visual inspection of the accessible interior and exterior surfaces of the shield building and verifying no apparent changes in appearance of the concrete surfaces or other abnormal degradation.

, in accordance with the Containment Leakage Rate Testing Program,

[illegible]

[illegible]

3/4.6 CONTAINMENT SYSTEMS

BASES

3/4.6.1 CONTAINMENT VESSEL

3/4.6.1.1 CONTAINMENT VESSEL INTEGRITY

CONTAINMENT VESSEL INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the site boundary radiation doses to within the limits of 10 CFR Part 100 during accident conditions.

[INSERT B]

which results from the limiting design basis loss of Coolant accident.

3/4.6.1.2 CONTAINMENT LEAKAGE

The limitations on containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the accident analyses at the peak accident pressure, P_a (39.6 psig). ~~As an added conservatism, the measured overall integrated leakage rate is further limited to $\leq 0.75\ L_a$ or $\leq 0.75\ L_t$ (as applicable) during performance of the periodic tests to account for possible degradation of the containment leakage barriers between leakage tests.~~ is performed in accordance with the Containment Leakage Rate Testing Program and is

The surveillance testing for measuring leakage rates ~~are~~ consistent with the requirements of Appendix "J" of 10 CFR 50, as modified by approved exemptions, ~~with the option of using any NRC approved method for performing the leak rate testing and calculating the results.~~ Option B and Regulatory Guide 1.163 Rev. Ø

3/4.6.1.3 CONTAINMENT AIR LOCKS

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on CONTAINMENT INTEGRITY and containment leak rate. Surveillance testing of the air lock seals provides assurance that the overall air lock leakage will not become excessive due to seal damage during the intervals between air lock leakage tests.

3/4.6.1.4 INTERNAL PRESSURE

The limitations on containment internal pressure ensure that 1) the containment structural is prevented from exceeding its design negative pressure differential with respect to the annulus atmosphere of 0.70 psi and 2) the containment peak pressure does not exceed the design pressure of 44 psig during steam line break accident conditions.

The maximum peak pressure obtained from a steam line break accident is 41.6 psig. The limit of 2.4 psig for initial positive containment pressure will limit the total pressure to 44.0 psig which is the design pressure and is consistent with the accident analyses.

3/4.6.1.5 AIR TEMPERATURE

The limitation on containment air temperature ensures that the containment vessel temperature does not exceed the design temperature of 264°F during LOCA conditions. The containment temperature limit is consistent with the accident analyses.

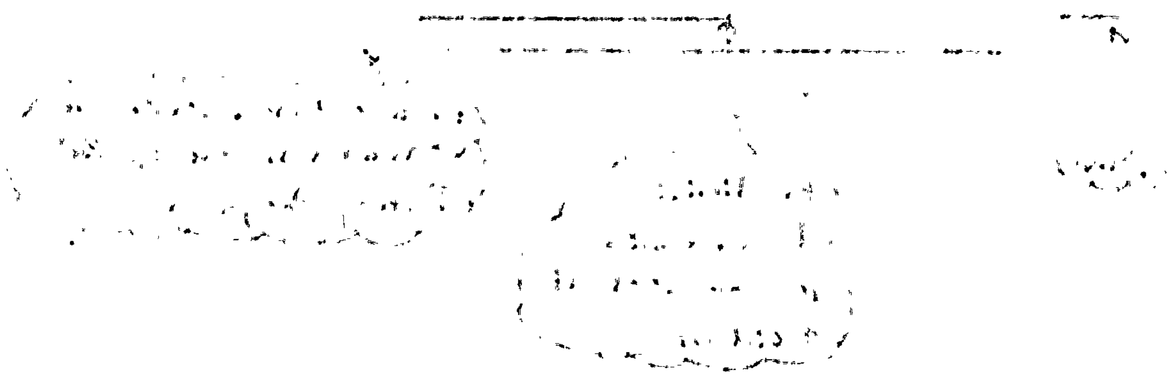
3/4.6.1.6 CONTAINMENT VESSEL STRUCTURAL INTEGRITY

This limitation ensures that the structural integrity of the containment steel vessel will be maintained comparable to the original design standards for the life of the facility. Structural integrity is required to ensure that the vessel will withstand the maximum pressure of ~~41.6~~ psig in the event of ~~a steam line break~~ accident. A visual inspection ~~in conjunction with Type A leakage test~~ is sufficient to demonstrate this capability.

39.6

the limiting design basis loss of Coolant accident

in accordance with the Containment Leakage Rate Testing Program



(2) conform to the guidance of Appendix I to 10 CFR Part 50, and
(3) include the following:

1) Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the ODCM,

2) A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census, and

3) Participation in a Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

[INSERT C]

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the NRC.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

[INSERT A Unit 1]

With the containment leakage rate exceeding the acceptance criteria of the Containment Leakage Rate Testing Program, within 1 hour initiate action to be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

[illegible]

[INSERT B Unit 1]

In accordance with Generic Letter 91-08, "Removal of Component Lists from Technical Specifications," the opening of locked or sealed closed containment isolation valves on an intermittent basis under administrative control includes the following considerations: (1) stationing an operator, who is in constant communication with the control room, at the valve controls, (2) instructing this operator to close these valves in an accident situation, and, (3) assuring that environmental conditions will not preclude access to close the valves and that this action will prevent the release of radioactivity outside the containment.

[INSERT C Unit 1]

h. Containment Leakage Rate Testing Program

A program to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50 Appendix J, Option B, as modified by approved exemptions. This program is in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," as modified by the following exception:

a. Bechtel Topical Report, BN-TOP-1 or ANS 56.8-1994 (as recommended by R.G. 1.163) will be used for type A testing.

The peak calculated containment internal pressure for the design basis loss of coolant accident, P_a, is 39.6 psig. The containment design pressure is 44 psig.

The maximum allowable containment leakage rate, L_a, at P_a, shall be 0.50% of containment air weight per day.

Leakage rate acceptance criteria are:

a. Containment leakage rate acceptance criterion is $\leq 1.0\ L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $< 0.60\ L_a$ for the Type B and C tests, $\leq 0.75\ L_a$ for Type A tests, and $\leq 0.27\ L_a$ for secondary containment bypass leakage paths.

b. Air lock testing acceptance criteria are:

1) Overall air lock leakage rate is $\leq 0.05\ L_a$ when tested at $\geq P_a$.

2) For the personnel air lock door seal, leakage rate is $< 0.01\ L_a$ when pressurized to $\geq P_a$.

3) For the emergency air lock door seal, leakage rate is $< 0.01\ L_a$ when pressurized to $\geq$ 10 psig.

The provisions of T.S. 4.0.2 do not apply to test frequencies in the Containment Leak Rate Testing Program.

The provisions of T.S. 4.0.3 are applicable to the Containment Leak Rate Testing Program.