

# CATEGORY 1

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BOHLKE, W.H.      Florida Power & Light Co.  
RECIP. NAME      RECIPIENT AFFILIATION  
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SUBJECT: Responds to NRC Bulletin 96-002, "Movement of Heavy Loads  
Over Spent Fuel, Over Fuel in Reactor Core or Over SR  
Equipment."

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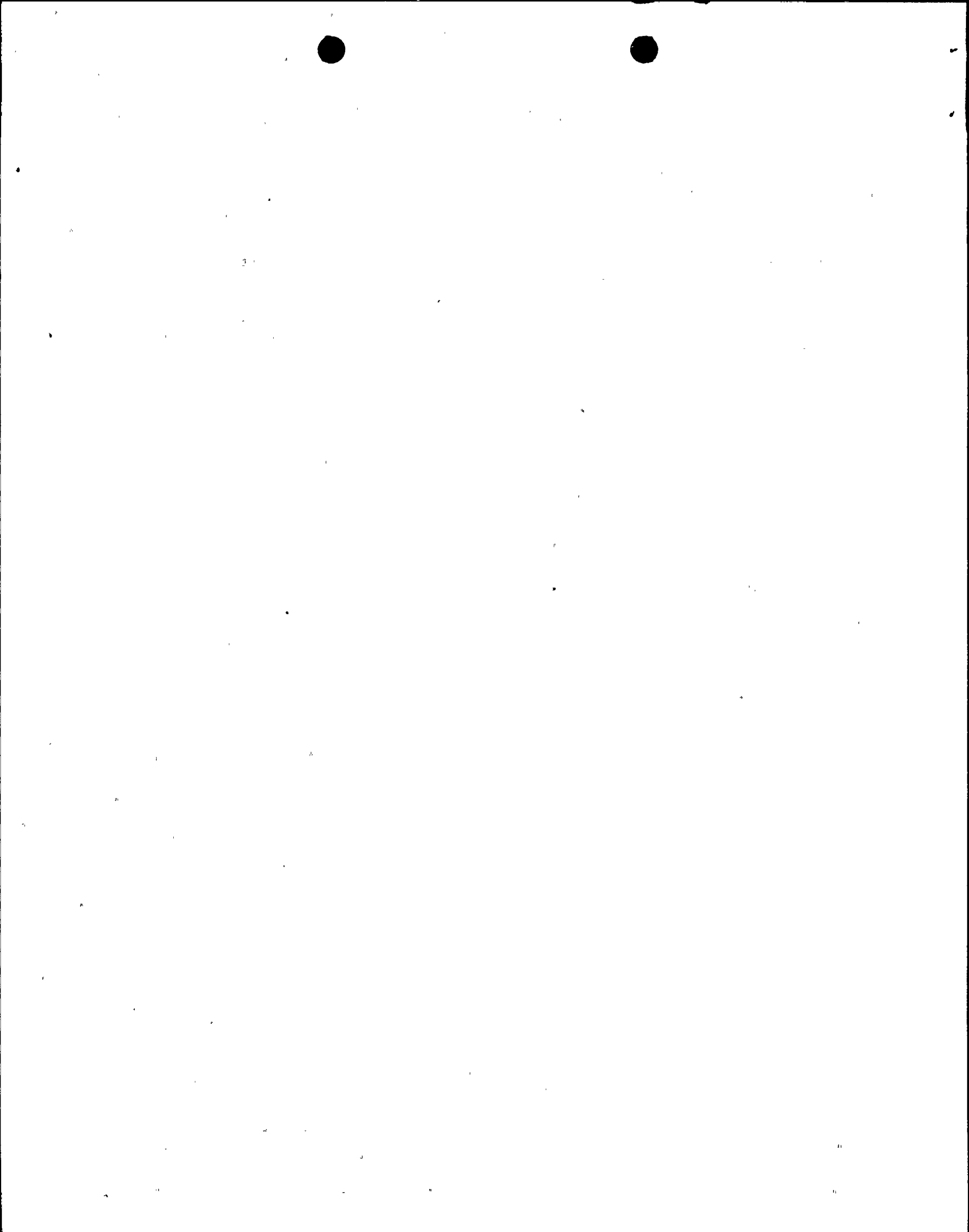
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Florida Power & Light Company, P.O. Box 128, Fort Pierce, FL 34954-0128

May 10, 1996

L-96-127  
10 CFR 50.4  
10 CFR 50.54(f)

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

RE: St. Lucie Units 1 and 2  
Docket Nos. 50-335 and 50-389  
NRC Bulletin 96-02 Response

Florida Power and Light Company (FPL) has completed the review requested by NRC Bulletin (NRCB) 96-02, *Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related Equipment*, for St. Lucie Units 1 and 2 and the requested response is attached.


The NRCB requested licensees to review their plans and capabilities for handling heavy loads in accordance with existing regulatory guidelines and within their licensing basis as previously analyzed in the final safety analysis report.

For licensees planning to implement activities involving the handling of heavy loads over spent fuel, fuel in the reactor core, or safety-related equipment within the next 2 years from the date of NRCB 96-02, April 11, 1996, the NRCB requested a report to be submitted within 30 days of the NRCB.

The attached information is provided pursuant to the requirements of Section 182a of the Atomic Energy Act of 1954, as amended, and 10 CFR 50.54(f).

Please contact us if there are any questions about this submittal.

Very truly yours,

  
W. H. Bohlke  
Vice President  
St. Lucie Plant

WHB/GRM

Attachments

cc: Stewart D. Ebnetter, Regional Administrator, Region II, USNRC  
Senior Resident Inspector, USNRC, St. Lucie Plant

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St. Lucie Units 1 and 2  
Docket Nos. 50-335 and 50-389  
L-96-127  
Bulletin 96-02 Response

STATE OF FLORIDA            )  
  )  
COUNTY OF ST. LUCIE        )            SS.

W. H. Bohlke being first duly sworn, deposes and says:

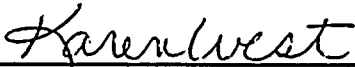
That he is Vice President, St. Lucie Plant for the Nuclear Division of Florida Power & Light Company, the Licensee herein;

That he has executed the foregoing document; that the statements made in this document are true and correct to the best of his knowledge, information and belief, and that he is authorized to execute the document on behalf of said Licensee.

  
\_\_\_\_\_  
W. H. Bohlke

STATE OF FLORIDA  
COUNTY OF St. Lucie

The foregoing instrument was acknowledged before me this 10<sup>th</sup> day of May, 1996 by W. H. Bohlke, who is personally known to me and who did take an oath.

  
\_\_\_\_\_  
KAREN WEST  
Name of Notary Public

My Commission expires 4-18-98  
Commission No. CC 359926



KAREN WEST  
MY COMMISSION # CC359926 EXPIRES  
April 18, 1998  
BONDED THRU TROY FAH INSURANCE, INC.



St. Lucie Units 1 and 2  
Docket Nos. 50-335 and 50-389  
L-96-127 Attachment  
Bulletin 96-02 Response

NRC REQUESTED ACTION

Review plans and capabilities for handling heavy loads while the reactor is at power (in all modes other than cold shutdown, refueling, and defueled) in accordance with existing regulatory guidelines. Determine whether the activities are within the licensing basis and if necessary, submit a license amendment request. Determine whether changes to Technical Specifications will be required in order to allow the handling of heavy loads (e.g., the dry storage canister shield plug and associated lifting devices) over fuel assemblies in the spent fuel pool.

FPL RESPONSE

Florida Power and Light Company (FPL) has completed the review requested by NRC Bulletin (NRCB) 96-02, *Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related Equipment*, for St. Lucie Units 1 and 2.

NRC REQUIRED RESPONSE 1

For licensees planning to implement activities involving the handling of a heavy load over spent fuel, fuel in the reactor core, or safety-related equipment within the next 2 years from the date of this bulletin, provide the following:

A report, to be submitted within 30 days of the date of this bulletin, that addresses the review of its plans and capabilities to handle heavy loads while the reactor is at power (in all modes other than cold shutdown, refueling, and defueled) in accordance with existing regulatory guidelines. The report should also indicate whether the activities are within the licensing basis and should include, if necessary, a schedule for submission of a license amendment request. Additionally, the report should indicate whether changes to Technical Specifications will be required.

FPL RESPONSE 1

Heavy load lifts are performed within the provisions of the St. Lucie Unit 1 and Unit 2 NUREG-0612 safety evaluation reports (SER) and applicable Technical Specification requirements. Heavy load lifts evaluated under NUREG-0612 Phase I are performed within the approved safe load paths or performed in accordance with the

St. Lucie Units 1 and 2  
Docket Nos. 50-335 and 50-389  
L-96-127 Attachment  
Bulletin 96-02 Response

special procedural guidelines of NUREG-0612. Specific deviations from the safe load paths require written alternative procedures which are approved by the Facility Review Group (FRG) as required by the NRC SER for NUREG-0612.

The alternative procedures for heavy load lifts are controlled by plant administrative procedure (AP)0010438, "Control of Heavy Loads," which requires prior approval by the FRG and the inclusion of the following information:

- a. Identification and weight of the load.
- b. Identification of the required lifting equipment.
- c. Inspections and acceptance criteria required before movement of the load.
- d. The steps, proper sequence, and safe load path to be followed in handling the load.
- e. The inclusion of special precautions. For lifts involving a mobile crane, the special precautions include the identification of maximum allowable load at the planned boom angle and extension and identification of the working radius needed for the lift.
- f. Verification that the planned lift is within the capacity of the crane in the configuration in which it will be used.

The following heavy load lifts are currently planned to be performed during the next 2 years while the applicable reactor is at power:

- A. The removal and reinstallation of the 1A and 2A intake cooling water (ICW) pumps for periodic preventive maintenance inspection/overhaul.

Although these pumps are not lifted over any "in-service" safety-related equipment, these lifts are performed in accordance with the plant administrative procedure (AP)0010438, "Control of Heavy Loads."

- B. The opening and closing of the 1A and 1B component cooling water (CCW) heat exchanger hinged channel head covers for tube cleaning and preventive maintenance.



St. Lucie Units 1 and 2  
Docket Nos. 50-335 and 50-389  
L-96-127 Attachment  
Bulletin 96-02 Response

This activity does not involve lifting of heavy loads over "in-service" safety-related equipment, but does require the use of a mobile crane to support the hinged channel head covers during the opening and closing operation and is performed in accordance with AP 0010438, "Control of Heavy Loads."

- C. Unit 1 turbine generator components/enclosures weighing less than five (5) tons are lifted over Unit 2 buried safety-related ICW piping while Unit 2 is at power. Unit 1 turbine generator components/enclosures can be lifted over Unit 2 buried safety-related conduits while Unit 2 is at power.

These heavy load lifts, which were evaluated as part of FPL NUREG-0612 response, were found to be acceptable based on physical separation of redundant conduits and/or adequate protection from a load drop and for the ICW piping a five ton load drop analysis.

- D. Unit 2 turbine generator components/enclosures are lifted over Unit 1 safety-related conduits and ICW piping while Unit 1 is at power.

These heavy load lifts, which were evaluated as part of the FPL NUREG-0612 response, were found to be acceptable based on physical separation of redundant conduits and ICW piping and/or adequate protection based on a load drop analysis.

NRC REQUIRED RESPONSE 2

For licensees planning to perform activities involving the handling of heavy loads over spent fuel, fuel in the reactor core, or safety-related equipment while the reactor is at power (in all modes other than cold shutdown, refueling and defueled) and that involve a potential load drop accident that has not previously been evaluated in the FSAR, submit a license amendment request in advance (6-9 months) of the planned movement of the loads so as to afford the staff sufficient time to perform an appropriate review.

FPL RESPONSE 2

St. Lucie Plant does not plan to perform any heavy load handling activities involving a potential load drop accident that has not previously been evaluated.

St. Lucie Units 1 and 2  
Docket Nos. 50-335 and 50-389  
L-96-127 Attachment  
Bulletin 96-02 Response

NRC REQUIRED RESPONSE 3

For licensees planning to move dry storage casks over spent fuel, fuel in the reactor core, or safety-related equipment while the reactor is at power (in all modes other than cold shutdown, refueling and defueled) include in item 2 above, a statement of the capability of performing the actions necessary for safe shutdown in the presence of radiological source term that may result from a breach of the dry storage cask, damage to the fuel, and damage to safety-related equipment as a result of a load drop inside the facility.

FPL RESPONSE 3

St. Lucie Plant does not plan to move dry storage casks over spent fuel, fuel in the reactor core, or safety-related equipment while the reactor is at power.

NRC REQUIRED RESPONSE 4

For licensees planning to perform activities involving the handling of heavy loads over spent fuel, fuel in the reactor core, or safety-related equipment while the reactor is at power (in all modes other than cold shutdown, refueling and defueled), determine whether changes to the Technical Specifications will be required in order to allow the handling of heavy loads (e.g., the dry storage canister shield plug) over fuel assemblies in the spent fuel pool and submit the appropriate information in advance (6-9 months) of the planned movement of the loads for NRC review and approval.

FPL RESPONSE 4

The planned heavy load lifts identified in item #1 above will not require changes to the Technical Specifications.