

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION FACIL: 50 AUTH.N LYONS,E SAGER,D RECIP.1	N NBR:9503200180 0-335 St. Lucie Pla AME AUTHOR A .E. Florida F .A. Florida F NAME RECIPIEN	DOC.DATE: Int, Unit 1 IFFILIATION Power & Lig Power & Lig T AFFILIAT	95/03/17 NOTARIZED , Florida Power & Li ht Co. ht Co. ION	ght Co.	DOCKET # 05000335	P
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SUBJECT	: LER 95-001-00:on for 1A3 4160 volt	950216,dur bus degra	ing portion of surve ded voltage relay,27	-5 relay	it .	I
د. •	failed to operate all equipment res	due to proponded as	ocedural deficiency. expected.W/950317 lt	Ops verifie r.	ed `	0
DISTRIB TITLE: !	UTION CODE: IE22T 50.73/50.9 Licensee	COPIES REC Event Rep	EIVED:LTR / ENCL / ort (LER), Incident	SIZE: $\leq$ Rpt, etc.	<u> </u>	R
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NOTE TO ALL "RIDS" RECIPIENTS:

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Florida Power & Light Company, P.O. Box 128, Fort Pierce, FL 34954-0128

March 17, 1995

L-95-088 10 CFR 50.73

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D. C. 20555

Re: St. Lucie Unit 1 Docket No. 50-335 Reportable Event: 95-001 Date of Event: February 16, 1995 <u>1A3 4160 Volt Bus Load Shed During Replacement</u> of Failed 2X-5 Relay Due to Procedural Deficiency.

The attached Licensee Event Report is being submitted pursuant to the requirements of 10 CFR 50.73 to provide notification of the subject event.

Very truly yours,

boal

D. A. Sader Vice President St. Lucie Plant

DAS/EJB

Attachment

cc: Stewart D. Ebneter, Regional Administrator, USNRC Region II Senior Resident Inspector, USNRC, St. Lucie Plant

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NRC FORM	1 366		U.S. NUCLEAR REGULATORY COMMISSION APPROVED BY ONB NO. 3150-0104 EXPIRES 5/31/95												
LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block) (See reverse for required number of digits/characters for each block)										LY WITH D.O HRS. MATE TO BRANCH MISSION, APERWORK ICE OF 503.					
FACILITY	FACILITY NAME (1)DOCKET NUMBER (2)PAGE (3)St. Lucie Unit 1050003351 OF 4														
TITLE (4) 1A3 4160 Volt Bus Load Shed During Replacement of Failed 2X-5 Relay due to															
EVEN	I DATE (3	<u>"</u>	r	LEK NUMBER (0)			KEPU			FACILI	TY NAME	11100 1111	DOCKET NUMBER		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	NUMBE	R	MONTH	DAY	YEAR		N/A				
02	16	95	95	001	0		03	17	95	FACILI	N/A		DUCKET	NUME	SER
OPERA"	TING	1	THIS RE	PORT IS SUBMITTE	D PURSU	IANT	TO THE	REQUIR	EMENTS	OF 10 C	FR §: (Check of	one or mor	e) (11	)	
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DUNG	FR I.	100	20.4	405(a)(1)(i)			50.36(0	)(1)			50.73(a)(2)()	/)	73.	.71(c	>
LEVEL	(10)	100	20.4	405(a)(1)(ii)		·	50.36(c	)(2)			50.73(a)(2)(v	/ii)	ITO	IER	
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			20.4	405(a)(1)(V)			50.73(8	)(2)(i	ii)		50.73(a)(2)()	()	NRC Fo	rm 30	56A)
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NAME	Edwa	ard	E. Ly	yons, Shift	t Te	chr	nical	Adv	visor	•	TELEPHONE NUX	HBER (Incl ) 465-	ude Ar 3550	ea Co X3	xde) 3151
			CON	PLETE ONE LINE FO	OR EACH	COH	PONENT	FAILURE	E DESCRI	BED IN	THIS REPORT (1	3)			
CAUSE	SYSTEM	CC	MPONENT	MANUFACTURER	REPOR TO N	TABL PRDS	E	0	CAUSE	SYSTE	COMPONENT	MANUFAC	TURER	REP TO	ORTABLE
x	EB		2	A348	2	ζ									
<u></u>	1	<u></u>	C11001 EME	NTAL DEDORT EXPE	CTED (1	4)				r		MONTH	DA	Î Y	YEAR
YES				CURNISSION DATE	\ \		x	40		5	UBMISSION DATE (15)				
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On 2/9/95 at 1840, Operations (Ops) was performing the monthly Channel Functional Test on the Undervoltage/Degraded voltage relaying scheme. During the portion of the surveillance test for the 1A3 4160 volt bus degraded voltage relaying, the 27-5 relay failed to operate. On 2/16/95 at 1100, Electrical Maintenance (EM) began the process of changing out the 2X-5 relay. At 1145, as a jumper was being removed, it inadvertently came in contact with terminal 2 on the 2X-6 relay. This made up the degraded voltage actuation circuit and the 1A3 4160 volt bus was shed. The 1A Emergency Diesel Generator started and loaded on to the 1A3 4160 volt bus. All equipment responded as expected. The root cause of the event was due to procedural deficiency. Corrective actions for the event are: 1) Ops verified all equipment responded as expected, 2) Ops restored power to the 1A3 4160 volt bus per ONOP 1-0910054, 3) EM completed the change out procedure for the 2X-5 relay, 4) Ops performed the surveillance test of the 1A3 4160 volt bus degraded voltage relaying with satisfactory results, 5) EM changed out all remaining 2X relays on both trains on Unit 1 and 6) A checklist will be developed to enhance St. Lucie Plant Policy 105 implementation.															

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NRC FORM 366 (5-92)

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NRC FORM 366A U.S. NU	CLEAR REGULATORY COMMISSION		APPROVED BY C	MB NO. 315	0-0104			
LICENSEE EVENT REPOR TEXT CONTINUATIO	T (LER) ON	ESTIMAT THIS IN FORWARD THE INI (MNBB 7 WASHING REDUCTI MANAGEM	ED BURDEN PER IFORMATION COLLI COMMENTS REGA FORMATION AND I 714), U.S. NUCL TON, DC 20555-0 ON PROJECT ENT AND BUDGET,	RESPONSE ECTION REQU RDING BURD RECORDS MAI EAR REGULAT (3150-0104) WASHINGTON	TO COMPLY WITH JEST: 50.0 HRS. IEN ESTIMATE TO NAGEMENT BRANCH ORY COMMISSION, O THE PAPERWORK O FFICE OF J. DC 20503.			
FACILITY HAME (1)	FACILITY NAME (1) DOCKET NUMBER (2)							
St. Lucie Unit 1	05000335	year 95	001	NUMBER 0	2 OF 4			
TEXT (If more space is required, use additional c	opies of NRC Form 366A) (17	7)		,				
DESCRIPTION OF THE EVENT								
On February 9, 1995 at 1840, Operat: required monthly Channel Functional scheme. During the portion of the s voltage relay, (EIIS:JE) 27-5 relay states that with the number of opera Channels, operation may proceed unt: test provided the inoperable channe. The 27-5 relay was placed in the tr jumper across terminals 1 and 2 on actuated by the 27-5 relay. The 2X actuated by the 27-6 relay, make up protection. With the jumper on the was 1 out of 1.	Test on the Undervo surveillance test for failed to operate. able Channels one les il performance of the l is placed in the t ipped condition at 1 the 2X-5 relay. The -5 relay, in conjunc the normal 2 out of 2X-5 relay the logi	the Te ltage/ r the Techn ss tha e next ripped 910 by 2X-5 tion w 2 log c for	chnical Spe Degraded vo 1A3 4160 vo ical Specif n the Total required o condition installing relay is a ith the 2X- ic for degr degraded vo	cification lt bus, ication Number hannel f within i f an elec timing f 6 relay aded vo bltage pr	lon elaying degraded 3.3.2.1 of functional 1 hour. ctrical relay that is ltage rotection			
Subsequent non-intrusive trouble she department revealed that the 2X-5 re was developed by Electrical Maintena Maintenance presented the procedure changed out on line but not without possibly the undervoltage/degraded to the confined wiring arrangement.	boting performed by elay had failed and ance to change the r to plant management some risk of load s voltage relaying sch	the el requir elay c stati hed of eme be	ectrical ma ed replacem ut on line. ng that the the 1A3 41 ing rendere	intenand ent. A Electra relay ( .60 volt ed inoper	ce procedure rical could be bus or rable due			
At this point plant management invoked St. Lucie Plant Policy PSL-105 which provides guidance for a Technical Subcommittee to be formed to review proposed actions that have risk to plant operations. The subcommittee included members from Technical Staff, Operations, Site Engineering, Electrical Maintenance and Plant Licensing. The subcommittee was tasked with reviewing the proposed procedure for impact on the plant, technical accuracy and to evaluate the consequences of various failure modes associated with the procedure. After a comprehensive review of the proposed procedure the subcommittee provided the following information to the Facility Review Group (FRG):								
The relay could be changed out on la recommendation was based on a detail similar event in the past in which undervoltage relay problem and the recommended the following compensation change out: 1) Electrical maintenan to commencement of work. 2) Independent Maintenance shall utilize "mini-gra ONOP 1-0910054, "LOSS OF A SAFETY R	ine with little addi led circuit analysis the 1A3 4160 volt bu plant remained on-li ory measures to be i nce supervision shal ndent verification s bber" type jumpers. ELATED A.C. BUS, " an	tional and a s was ne. Th mpleme l cond hall b 4) Th d cons	risk of a historical load shed d load shed d inted prior huct a tailk be performed le operating ider potent	plant t: review to to au to the : coard mee . 3) E g crew sh tal imposi	rip. This of a n relay eting prior lectrical hall review act on unit			

and appropriate contingency actions. 5) The operating crew shall verify CEDM MG set (EIIS:AA) load is shared equally (+/- 20 amps). 6) Electrical maintenance shall insulate exposed leads. 7) Ensure spare fuses for the undervoltage relaying circuit were available at the work location in the event that the D.C. control power was inadvertently shorted.

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The FRG concluded that the relay change out could proceed without imposing undue safety risk.

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NRC FORM 366A (5-92)	U.S. NUCLEAR RE	GULATORY COMMISSION	APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95						
	LICENSEE EVENT REPORT (LE TEXT CONTINUATION	ESTIMATED BURDEN PER RESPONSE TO COMPLY WI THIS INFORMATION COLLECTION REQUEST: 50.0 HR FORWARD COMMENTS REGARDING BURDEN ESTIMATE THE INFORMATION AND RECORDS MANAGEMENT BRAN (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSIO WASHINGTON, DC 20555-0001, AND TO THE PAPERWO REDUCTION PROJECT (3150-0104), OFFICE MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.							
	FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6	)	PAGE (3)			
	St. Lucie Unit 1		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER				
	·····	05000335	95	001	0	3 OF 4			
TEXT (If more	space is required, use additional copies of	NRC Form 366A) (17	<u>~</u>	v					

DESCRIPTION OF THE EVENT (continued)

On 2/16/95 at 1100, following extensive tailboards with Operations and Maintenance personnel, Electrical Maintenance began the process of changing out the 2X-5 relay. Steps 1 through 34 of a 38 step procedure were completed without incident. Step 35 called for the jumper that was keeping the 27-5 relay in trip to be removed to allow the surveillance test to be completed. This jumper was from terminal 1 on the 2X-1 relay to terminal 1 on the 2X-6 relay. At 1145, after the 2X-5 relay was replaced, as the jumper was being removed from terminal 1 on the 2X-6 relay it inadvertently came in contact with terminal 2 on the 2X-6 relay. This made up the degraded voltage actuation circuit and the 1A3 4160 volt bus was shed. The 1A Emergency Diesel Generator (EIIS:EK) started and loaded on to the 1A3 4160 volt bus. All equipment responded as expected. The event had minimal impact on the plant due to the compensatory measures recommended by the subcommittee being in place. Offsite power was restored to the 1A3 4160 volt bus at 1207 and the channel functional test for degraded voltage was completed at 1247.

## CAUSE OF THE EVENT

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The root cause of the event was due to procedural deficiency. The Technical subcommittee was asked to critically reevaluate the event. The team came up with the following considerations for future Technical subcommittee reviews: Provide more detailed review of the sequence in which the ends of jumpers are landed and removed to minimize the impact of inadvertent contact with adjacent terminals, evaluate consequences of shorting of adjacent wires and other wires which could be easily shorted, review specific methods of insulating close proximity adjacent terminals and possibly utilize jumpers with series toggles switches to open circuit the jumper prior to disconnection.

## ANALYSIS OF THE EVENT

This event is reportable under 10 CFR 50.73.a.2.iv as "any event or condition that resulted in manual or automatic actuation of any engineered safety feature, including the Reactor Protective System (RPS) (EIIS:JC)."

Only one train of safety related components was temporarily de-energized during the event. The 1A EDG started and loaded onto the safety related bus as designed. The redundant safety related train was available for service and the plant remained in a stable condition throughout the entire event. Offsite power was always available to both electrical trains during this event. Thus, the health and safety of the public were not affected at any time during the event.

FPL supplied failed DSC relays to National Technical Systems (NTS) so that they could be analyzed and a cause for the failures could be determined. NTS concluded from visual inspections and unit functional testing that the failures were caused by natural aging of the units since installation, in the environment in which the relays were being installed.

This event is not applicable to both units as Unit 2 does not use Agastat DSC relays in the degraded grid circuit.

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NRC FORM 366A U. (5-92)	S. NUCLEAR RE	GULATORY COMMISSION	APPROVED BY OMB NO. 3150-0104 ' EXPIRES 5/31/95						
LICENSEE EVENT RE TEXT CONTINU	ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (HNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND RUDGET WASHINGTON DC 20503								
FACILITY NAME (1)	FACILITY NAME (1) DOCKET NUMBER (2)								
St. Lucie Unit 1		05000335	year 95	SEQUENTIAL NUMBER	REVISION NUMBER O	40	F 4		
TEXT (If more space is required, use additic	onal copies of	NRC Form 366A) (17	/ <u></u> /		l	l			
CORRECTIVE ACTIONS	•	*		۲					
1) Operations verified all equip	oment resp	onded as expec	ted.						
2) Operations restored power to	 the 1A3 4	160 volt bus p	er ONO	P 1-0910054	•				
3) Flectrical Maintenance comol	eted the c	hange out proc	edure	for the 2X-	5 relav	,			
<ul> <li>4) Operations performed the survivor relaying with satisfactory relaying</li> </ul>	veillance esults.	test of the 1A	3 4160	volt bus d	legraded	volta	ge		
5) Electrical Maintenance change	ed out all	remaining 2X :	relays	on both tr	ains on	Unit	1.		
6) A checklist will be developed	d to enhan	ce St. Lucie P	lant P	olicy 105 i	mplement	cation			
ADDITIONAL_INFORMATION									
Component Failures: Amerace Con Model DSCX	rp Agastat X0125DP	Time Delay Re	lay						
Previous_Similar_Events									
LER 335-94-002 "Inadvertent Load Inadequacy"	d Shed of	the 1A3 4160 V	olt Bu	s due to Pr	ocedura.	1			
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