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SUBJECT: Forwards info re acceptance criteria for ECCS for light water nuclear power reactors.

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February 28, 1995

L-95-071  
10 CFR 50.46

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

Re: St. Lucie Units 1 and 2  
Docket Nos. 50-335 and 50-389  
Acceptance Criteria for Emergency Core Cooling  
Systems for Light Water Nuclear Power Reactors:  
10 CFR 50.46 Annual Report

Pursuant to 10 CFR 50.46(a)(3)(ii), the nature of any change to or error discovered in the evaluation models for Emergency Core Cooling Systems (ECCS) or in the application of such models that affect the fuel cladding temperature calculations for St. Lucie Units 1 and 2, are reported in ATTACHMENT 1. The resultant effect on the limiting ECCS analysis for each unit is also addressed. The data interval for the report is from January 1 through December 31, 1994.

Should there be any questions, please contact us.

Very truly yours,

D. A. Sager  
Vice President  
St. Lucie Plant

DAS/RLD

cc:

Stewart D. Ebnetter, Regional Administrator, Region II, USNRC  
Senior Resident Inspector, USNRC, St. Lucie Plant

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ECCS analyses for St. Lucie Units 1 and 2 are performed by Siemens Power Corporation (SPC) and Asea Brown Boveri-Combustion Engineering (ABB-CE). Identification of the vendor performing the analysis of record for Small Break Loss of Coolant Accidents (SBLOCA) and Large Break Loss of Coolant Accidents (LBLOCA), and appropriate data involving each analysis during the period January 1, 1994 through December 31, 1994, follows.

1. ST. LUCIE UNIT 1

LBLOCA, Limiting ECCS Analysis, SPC (1/1/94 - 12/31/94): Changes were not made to nor were errors discovered in the evaluation model, or in its application, that would affect the calculated peak cladding temperature (PCT) of the St. Lucie Unit 1 limiting ECCS analysis of record.

SBLOCA: ABB-CE (1/1/94 - 11/24/94): One error was discovered in the ABB-CE evaluation model for SBLOCA which involved the method used to account for thermal expansion of zirconium cladding. The effect on calculated PCT was less than 1 °F.

SBLOCA: SPC (11/25/94 - 12/31/94): The St. Lucie Unit 1 SBLOCA was analyzed by SPC as part of a licensing action to support operation with up to 25% (average) steam generator tube plugging. The evaluation model was found acceptable by the NRC and the analysis was approved by issuance of Amendment No. 130 (11/25/94) to facility operating license DPR-67. SPC's evaluation of SBLOCA thus became the Unit 1 analysis of record for such transients. The calculated PCT is 1846 °F, which is 111 °F less than the previous analysis of record.

2. ST. LUCIE UNIT 2

LBLOCA, Limiting ECCS Analysis, ABB-CE (1/1/94 - 12/31/94): Two errors were discovered in the ABB-CE evaluation model for LBLOCA. One error involved the method used to account for thermal expansion of zirconium cladding. The other error involved default values which could be used for certain input parameters to the code. Neither of these errors affected the calculated PCT of the St. Lucie Unit 2 limiting ECCS analysis of record.

SBLOCA, ABB-CE (1/1/94 - 12/31/94): One error was discovered in the ABB-CE evaluation model for SBLOCA which involved the method used to account for thermal expansion of zirconium cladding. The effect on calculated PCT was less than 1 °F.