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SUBJECT: Responds to request for addl info re Millstone Nuclear Power Station Unit 2 for GL 92-001.

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NORTHEAST NUCLEAR ENERGY COMPANY

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January 4, 1993

Docket No. 50-336  
B14329

Re: Generic Letter 92-01,  
Revision 1

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Gentlemen:

Millstone Nuclear Power Station, Unit No. 2  
Reactor Vessel Structural Integrity, 10CFR50.54(f)  
(Generic Letter 92-01, Revision 1)  
Response to Request for Additional Information

In a letter dated March 6, 1992,<sup>(1)</sup> the NRC Staff issued Generic Letter 92-01, Revision 1, "Reactor Vessel Structural Integrity, 10CFR50.54(f)." A response to Generic Letter 92-01 was provided on July 6, 1992.<sup>(2)</sup> Detailed information related to the subject was provided in Table 1 of the July 6, 1992, letter. Based on telephone discussions held with the NRC Staff on December 16 and December 18, 1992, Northeast Nuclear Energy Company (NNECO) is hereby providing additional information related to the upper shelf energy of the Millstone Unit No. 2 reactor vessel, as requested.

In accordance with 10CFR50.54(f), NNECO, on behalf of Millstone Unit No. 2, hereby provides the attached additional information to supplement the information provided by NNECO in our July 6, 1992, letter.

- (1) J. G. Partlow letter to All Holders of Operating Licenses or Construction Permits for Nuclear Power Plants (Except Yankee Atomic Electric Company, Licensee for the Yankee Nuclear Power Station), "Reactor Vessel Structural Integrity, 10CFR50.54(f), (Generic Letter 92-01, Revision 1)," dated March 6, 1992.
- (2) J. F. Opeka letter to the U.S. Nuclear Regulatory Commission, "Reactor Vessel Structural Integrity, 10CFR50.54(f), (Generic Letter 92-01, Revision 1)," dated July 6, 1992.

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U.S. Nuclear Regulatory Commission  
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January 4, 1993

If you have any questions, please contact us.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

FOR: J. F. Opeka  
Executive Vice President

BY:   
E. A. DeBarba  
Vice President

Enclosure

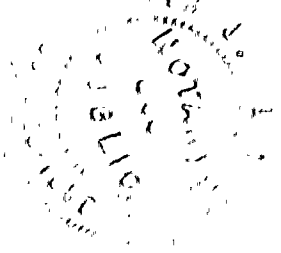
cc: T. T. Martin, Region I Administrator  
G. S. Vissing, NRC Project Manager, Millstone Unit No. 2  
P. D. Swetland, Senior Resident Inspector, Millstone Unit Nos. 1, 2,  
and 3

Subscribed and sworn to before me

this 4<sup>th</sup> day of January, 1993

  
Notary Public

Date Commission Expires: 3/31/95



Docket No. 50-336  
B14329

Millstone Nuclear Power Station, Unit No. 2

Additional Information for the  
Response to Generic Letter 92-01

January 1993

Millstone Nuclear Power Station, Unit No. 2  
Additional Information for the  
Response to Generic Letter 92-01

TOPIC: Additional information related to the Upper Shelf Energy of the Millstone Nuclear Power Station, Unit No. 2 reactor vessel, as requested by the Nuclear Regulatory Commission.

RESPONSE:

NNECO has developed the following plan to address the potential end-of-life low Upper Shelf Energy (USE) issues identified during the NRC review of the Millstone Unit No. 2 submittal on Generic Letter 92-01, Revision 1, "Reactor Vessel Structural Integrity, 10CFR50.54(f)." The plan can be summarized as follows:

1. The ABB/CE Owner's Group (CEOG) has performed a bounding low USE analysis in accordance with the ASME Appendix X methodology for all of the ABB/CE Nuclear Steam System Supply (NSSS) plants. The preliminary version of this analysis, which was submitted to the NRC on November 13, 1992, demonstrated the required margins of safety for materials with USE as low as 45 ft-lbs. Since the end-of-life USE of the Millstone Unit No. 2 reactor vessel is not predicted to fall below 45 ft-lbs, the CEOG report is considered applicable to the Millstone Unit No. 2 vessel.
2. The final version of the above ASME Appendix X evaluation is expected to be submitted for NRC review by April 30, 1993, and it is expected to confirm the results reached during the preliminary evaluation.
3. Material from the vessel plates, which could be predicted to fall below the 50 ft-lbs requirement, has recently been located in the vessel archives. The pedigree of this material is currently being evaluated to determine the feasibility of testing these materials to obtain the actual, unirradiated, transverse USE. This test program is expected to be completed April 30, 1993.

In summary, the USE of the Millstone Unit No. 2 reactor vessel is currently above 50 ft-lbs, as required by 10CFR50, Appendix G, and it is not expected to decrease below 50 ft-lbs prior to the completion of the material testing program currently under evaluation. Furthermore, the CEOG evaluations performed to date, demonstrate that the ASME Appendix G margins will continue to be satisfied to USE values as low as 45 ft-lbs, which bound the Millstone Unit No. 2 worst case, end-of-life USE values.



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