

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 18

TO FACILITY OPERATING LICENSE NO. NPF-16

FLORIDA POWER & LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNIT NO. 2

DOCKET NO. 50-389

INTRODUCTION

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By application dated June 17, 1983, the Florida Power and Light Company (FP&L) requested various changes to Appendix A of Facility Operating License NPF-16 for the St. Lucie Plant, Unit No. 2. The proposed amendment consists of a revision to the steam generator pressure/temperature limits, as well as the inclusion of several changes considered to be administrative and/or editorial in nature. The specific changes are discussed and evaluated below.

Technical Specification (TS) 3.7.2, Surveillance Requirement TS 4.7.2, and the applicable Bases, B 3/4.7.2, would be revised such that only the secondary side coolant temperature in the steam generators would be required to be greater than 100° F when the pressure of the secondary coolant in the steam generator is greater than a reduced pressure of 200 psig. The basis for this change is to ensure that pressure induced stresses in the steam generator do not exceed the fracture toughness stress limits of the steam generator materials.

The pressure/temperature limits for the steam generators are reviewed in accordance with the fracture toughness requirements of Standard Review Plan SRP 5.4.2.1, "Steam Generator Materials." This SRP requires that the primary side steam generator materials must meet the fracture toughness requirements of Appendix G, 10 CFR Part 50, and that the secondary side steam generator materials must meet the fracture toughness requirements of Section III, Subarticle NC 2300 of the ASME Code. The Edition and Addenda of the ASME Code applicable to the secondary side materials is described in Section 50.55a, 10 CFR Part 50, as the 1980 Edition and Addenda through the Winter 1980 Addenda. Pressure/temperature limits, which meet these code and regulatory requirements, will ensure that pressure induced stresses in the steam generator do not exceed the fracture toughness of the steam generator materials. The proposed pressure/temperature limits are based on a steam generator reference temperature nil ductility transition (RT_{NDT}) of 20°F.

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The staff's review indicates that the pressure/temperature limits proposed by the licensee meet the fracture toughness requirements of SRP 5.4.2.1 and, therefore, the staff finds the proposed changes acceptable.

TS 3.4.1.1, which refers to the startup and power operation of the reactor coolant loops and coolant circulation, would be revised such that the footnote associated with the <u>APPLICABILITY</u> statement would no longer read, "See Special Test Exceptions 3.10.3 and 3.10.6", but would instead read, "See Special Test Exception 3.10.3". The staff finds this change acceptable, as our review concludes that there is no Special Test Exception 3.10.6.

TS 3.7.11.1, which refers to the fire suppression water system of the fire suppression systems, would be revised such that TS 3.7.11.1c would no longer read, "per Specifications 3.7.11.2, 3.7.11.4 and 3.7.11.5", but would instead read, "per Specifications 3.7.11.2, 3.7.11.3 and 3.7.11.4". The staff finds this change acceptable, as our review concludes that there is no Specification 3.7.11.5 and that the inclusion of Specification 3.7.11.3 is correct.

FP&L proposed to revise TS 6.9.1.10, "Semiannual Radioactive Effluent Release Report," such that the words "first half year" would be deleted from the first line of the double asterisk footnote. This revision was proposed prior to the implementation of Amendment 13. Amendment 13 revised the specification number from 6.9.1.10 to 6.9.1.7. The staff finds this change acceptable, as it serves to clarify the statement and is editorial in nature.

FP&L proposed to revise the main feedwater line isolation valves' closure times referred to in TS 4.7.1.6. Revised closure times were incorporated into TS 4.7.1.6 by Amendment No. 8 issued on November 9, 1984. Therefore, the staff takes no action on this item, as it has been previously addressed.

ENVIRONMENTAL CONSIDERATION

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This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes an inspection or a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). This amendment also involves changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, with respect to these items, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

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CONCLUSION

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We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 11, 1987

Principal Contributors: B. Elliot R. Perfetti





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