November 25, 1986

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Docket No. 50-335

Mr. C. O. Woody
Group Vice President
Nuclear Energy
Florida Power & Light Company
P. O. Box 14000
Juno Beach, Florida 33408

Dear Mr. Woody:

SUBJECT: REANALYSIS OF RCP SEIZED ROTOR AND LOSS OF ALL NON-EMERGENCY AC POWER

In conducting our review of your August 31, 1982 and September 11, 1984 submittals relating to the above subject at the St. Lucie Plant, Unit No. 1, we have determined that we will need the additional information identified in the enclosure to continue our review.

In order for us to maintain our review schedule, your response is required within 30 days of receipt of this letter.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than 10 respondents; therefore OMB clearance is not required under P.L. 96-511.

Please contact me at (301) 492-8007 if you have any questions concerning this letter.

Sincerely,

/S/

E. G. Tourigny, Project Manager
PWR Project Directorate #8
Division of PWR Licensing-B

Enclosure:
Request for Additional Information

cc: w/enclosure
See next page

PBD-8:	PBD-8:	RSB	PBD-8:
PKreutzer	EGTourigny:jch	ELantz	AThadani
10/24/86	10/28/86	10/25/86	10/28/86
TAC#48795	11/24/86		

Mr. C. O. Woody
Florida Power & Light Company

St. Lucie Plant

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Request for Additional Information
for St. Lucie Unit No. 1
Reanalysis of RCP Seized Rotor and Loss of All Non-Emergency AG Power

References:

1. Letter from C. C. Nelson, USNRC, to R. E. Uhrig, Florida Power & Light Company, November 23, 1981.
2. Letter from R. A. Clark, USNRC, to R. E. Uhrig, Florida Power & Light Company, April 26, 1982.
3. Letter from J. W. Williams, Jr., Florida Power & Light Company, to J. R. Miller, USNRC, September 11, 1984.
4. Letter from D. E. Sells, USNRC, to C. O. Woody, Florida Power and Light Company, January 15, 1986.

In the Safety Evaluation for St. Lucie, Unit 1, License Amendment No. 48 (Reference 1), which authorized an increase in the licensed power from 2560 to 2700 MWth, it was found that three items needed further attention. One of these items was closed out via Reference 2; two items remained open.

The first open item was an analysis of the loss of non-vital AC power taking into consideration the single failure criterion. The second open item was an analysis of the seized reactor coolant pump rotor event taking into consideration loss of offsite power and the single failure criterion.

Florida Power & Light Company (FPLC) reanalyzed these two postulated events and submitted the results to the NRC on August 31, 1982. Following requests for additional information, FPLC made a second submittal at the end of 1983 and a third on September 11, 1984 (Reference 3).

After reviewing these submittals, the staff finds that information provided concerning the flow of auxiliary feedwater during these two postulated events is still unclear. FPLC's response to question 3 requires further clarification. The final two sentences of the licensee's response to question 3c are:

> "As a result, both steam generators depressurize virtually at the same rate, showing no apparent asymmetry. However, since the affected steam generator discharges more steam through the stuck opened ADV than the unaffected steam generator due to its proximity to that valve, its level reaches the AFW actuation setpoint; initiating the delivery of the AFW following a 2 minute delay time."

These two sentences appear to be contradictory. If both steam generators depressurize at virtually the same rate, how could more steam be discharged from one than the other? Wouldn't the water levels in the two steam generators have to be the same? FPLC should explain this apparent discrepancy. Specifically, FPLC should provide time-dependent plots of the various important parameters, e.g., steam generator pressure and level, main and auxiliary feedwater flow and steam flow, in order to support the response.

Lastly, Reference 4 approved an AFW delivery time delay of greater than two minutes. FP&L needs to address the different assumptions on feedwater delivery time and describe how the reference 4 assumption affects the conclusions in the analyses associated with the above two open items.