

UNITED STATES NUCLEAR REGULATORY COMMISSION VASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET .NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73 License No. DPR-67

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, (the licensee) dated May 30, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Ashok C. Thadani, Director PWR Project Directorate #8 Division of PWR Licensing-B

Attachment: Changes to the Technical Specifications

Date of Issuance: May 30, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 73

TO FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove Page

<u>Insert Page</u>

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REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- b. If the inservice inspection of a steam generator conducted in accordance with Table 4.4-2 requires a third sample inspection whose results fall in Category C-3, the inspection frequency shall be reduced to at least once per 20 months. The reduction in inspection frequency shall apply until a subsequent inspection demonstrates that a third sample inspection is not required.
- c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-2 during the shutdown subsequent to any of the following conditions.
 - 1. Primary-to-secondary tubes leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.6.2,
 - 2. A seismic occurrence greater than the Operating Basis Earthquake,
 - 3. A loss-of-coolant accident requiring actuation of the engineered safeguards, or
 - 4. A main steam line or feedwater line break.

4.4.5.4 <u>Acceptance Criteria</u>

- a. As used in this Specification:
 - 1. <u>Imperfection</u> means an exception to the dimensions, finish or contour of a tube from that required by fabrication drawings or specifications. Eddy-current testing indications below 20% of the nominal tube wall thickness, if detectable, may be considered as imperfections.
 - 2. <u>Degradation</u> means a service-induced cracking, wastage, wear or general corrosion occurring on either inside or outside of a tube.
 - 3. <u>Degraded Tube</u> means a tube containing imperfections >20% of the nominal wall thickness caused by degradation.
 - 4. <u>% Degradation</u> means the percentage of the tube wall thickness affected or removed by degradation.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- 5. <u>Defect</u> means an imperfection of such severity that it exceeds the plugging limit. A tube containing a defect is defective. Any tube which does not permit the passage of the eddycurrent inspection probe shall be deemed a defective tube.
- 6. Plugging Limit means the imperfection depth at or beyond which the tube shall be removed from service because it may become unserviceable prior to the next inspection and is equal to 40%* of the nominal tube wall thickness.
- 7. Unserviceable describes the condition of a tube if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break as specified in 4.4.5.3.c, above.
- 8. <u>Tube Inspection</u> means an inspection of the steam generator tube from the point of entry (hot leg side) completely around the U-bend to the top support of the cold leg.
- b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.4-2.

4.4.5.5 Reports

- a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.
- b. The complete results of the steam generator tube inservice inspection shall be included in the Annual Operating Report for the period in which this inspection was completed. This report shall include:
 - 1. Number and extent of tubes inspected.
 - 2. Location and percent of wall-thickness penetration for each indication of an imperfection.
 - Identification of tubes plugged.

^{*}This 40% plugging limit is not applicable during the cycle 7 operation up to June 30, 1986. If at any time during this period the unit enters any Modes other than Modes 1 and 2, or Mode 3 for greater than 24 hours, the unit shall be placed in cold shutdown and the tubes with indications greater than 40% through-wall penetration shall be removed from service prior to exceeding 200°F.