ATTACHMENT 1

CHANGES TO TECHNICAL SPECIFICATIONS

ST. LUCIE UNIT 1

DPR-67

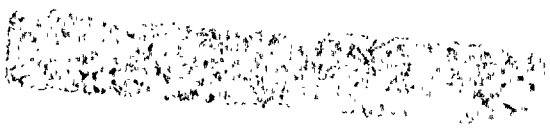

TABLE 3.3-11

ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	TOTAL NO. OF CHANNELS	MINIMUM CHANNELS OPERABLE	ACTION
1. Pressurizer Water Level	3	1	1
2. Auxiliary Feedwater Flow Rate	1/pump	1/pump	1
3. RCS Subcooling Margin Monitor	1	1	1
4. PORV Position Indicator Acoustic Flow Monitor	1/valve	1/valve	2
5. PORV Block Valve Position Indicator	1/valve	1/valve	2
6. Safety Valve Position Indicator	1/valve	1/valve	3
7. INCORE THERMOCOUPLES	4/CORE QUADRANT	2/Core Quadrant	1
8. CONTAINMENT Sump Water Level (Narrow Range)	1*	1*	4, 5
9. CONTAINMENT Sump Water Level (WIDE RANGE)	2	1	4, 5
10 Reactor Vessel Level Monitoring System	2**	1**	4, 5
11 CONTAINMENT Pressure	2	1	1

* THE NON-SAFETY GRADE CONTAINMENT SUMP WATER LEVEL INSTRUMENT MAY BE SUBSTITUTED

** DEFINATION OF Operable : A CHANNEL IS COMPOSED of (8) eight SENSORS IN A PROBE, OF WHICH (4) four SENSORS MUST BE OPERABLE.

TABLE 3.3-11 (Continued)

ACTION STATEMENTS

ACTION 1 - With the number of OPERABLE channels less than required by Table 3.3-11, either restore the inoperable channel(s) to OPERABLE status within 30 days or be in HOT STANDBY within the next 12 hours.

ACTION 2 - With position indication inoperable, restore the inoperable indicator to OPERABLE status or close the associated PORV block valve and remove power from its operator within 48 hours or be in HOT STANDBY within the next 6 hours.

ACTION 3 - With any individual valve position indicator inoperable, obtain quench tank temperature, level and pressure information once per shift to determine valve position.

ACTION 4 - With the number of OPERABLE Channels one less than the Total Number of Channels shown in Table 3.3-11, either restore the inoperable channel to OPERABLE status within 7 days if repairs are feasible without shutting down or prepare and submit a Special Report to the commission pursuant to the specification 6.9.2 within 30 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

ACTION 5 - With the number of OPERABLE Channels less than the Minimum Channels OPERABLE requirements of Table 3.3-11, either restore the inoperable channel(s) to OPERABLE status within 48 hours if repairs are feasible without shutting down or:

1. Initiate an alternate method of monitoring the reactor vessel inventory; and

2. Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 30 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status; and

3. Restore the Channel to OPERABLE status at the next scheduled refueling.

TABLE 4.3-7

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1. Pressurizer Water Level	M	R
2. Auxiliary Feedwater Flow Rate	M	R
3. Reactor Coolant System Subcooling Margin Monitor	M	R
4. PORV Position Indicator	M	R
5. PORV Block Valve Position Indicator	M	R
6. Safety Valve Postition Indicator	M	R
7. Incore Thermocouples	M	R
8. Containment Sump Water Level (Narrow Range)	M	R
9. Containment Sump Water Level	M	R
10. Reactor Vessel Level Monitoring System	M	R
11. Containment Pressure	M	R

ATTACHMENT 2

CHANGES TO TECHNICAL SPECIFICATIONS

ST. LUCIE UNIT 2

NPF-16

INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.6 The accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

a.* With the number of OPERABLE accident monitoring channels less than the Required Number of Channels shown in Table 3.3-10, either restore the inoperable channel to OPERABLE status within 7 days, or be in HOT SHUTDOWN within the next 12 hours.

b.* With the number of OPERABLE accident monitoring channels less than the Minimum Channels OPERABLE requirements of Table 3.3-10; either restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

Add Actions c & d (Attached) →

e f. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.6 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

* ACTION STATEMENTS DO NOT APPLY TO REACTOR VESSEL LEVEL MONITORING SYSTEM, CONTAINMENT SUMP WATER LEVEL (NARROW RANGE) AND CONTAINMENT SUMP WATER LEVEL (WIDE RANGE) INSTRUMENTS

** ACTION STATEMENTS APPLY ONLY TO REACTOR VESSEL LEVEL MONITORING SYSTEM, CONTAINMENT SUMP WATER LEVEL (NARROW RANGE) AND CONTAINMENT SUMP WATER LEVEL (WIDE RANGE) INSTRUMENTS.

ADD THESE ACTION STATEMENTS TO 3/4 3-41

C.** With the number of OPERABLE Channels one less than the Total Number of Channels shown in Table 3.3-11, either restore the inoperable channel to OPERABLE status within 7 days if repairs are feasible without shutting down or prepare and submit a Special Report to the commission pursuant to the specification 6.9.2 within 30 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

D.** With the number of OPERABLE Channels less than the Minimum Channels OPERABLE requirements of Table 3.3-11, either restore the inoperable channel(s) to OPERABLE status within 48 hours if repairs are feasible without shutting down or:

1. Initiate an alternate method of monitoring the reactor vessel inventory; and

2. Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 30 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status; and

3. Restore the Channel to OPERABLE status at the next scheduled refueling.

TABLE 3.3-10

ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	REQUIRED NUMBER OF CHANNELS	MINIMUM CHANNELS OPERABLE
1. Containment Pressure	2	1
2. Reactor Coolant Outlet Temperature - T_{Hot} (Narrow Range)	2	1
3. Reactor Coolant Inlet Temperature - T_{Cold} (Wide Range)	2	1
4. Reactor Coolant Pressure - Wide Range	2	1
5. Pressurizer Water Level	2	1
6. Steam Generator Pressure	2/steam generator	1/steam generator
7. Steam Generator Water Level - Narrow Range	1/steam generator	1/steam generator
8. Steam Generator Water Level - Wide Range	1/steam generator*	1/steam generator*
9. Refueling Water Storage Tank Water Level	2	1
10. Auxiliary Feedwater Flow Rate (Each pump)	1/pump*	1/pump*
11. Reactor Cooling System Subcooling Hargin Monitor	2	1
12. PORV Position/Flow Indicator	2/valve***	1/valve **
13. PORV Block Valve Position Indicator	1/valve**	1/valve**
14. Safety Valve Position/Flow Indicator	1/valve***	1/valve***
15. Containment Sump Water Level (Narrow Range)	1****	1****
16. Containment Water Level (Wide Range)	2	1
17. Incore Thermocouples	4/core quadrant	2/core quadrant
18. Reactor Vessel Level Monitoring System	2 *****	1 *****

*These corresponding instruments may be substituted for each other.

**Not required if the PORV block valve is shut and power is removed from the operator.

***If not available, monitor the quench tank pressure, level and temperature, and each safety valve/PORV discharge piping temperature at least once every 12 hours.

****The non-safety grade containment sump water level instrument may be substituted.

***** DEFINATION OF OPERABLE: A CHANNEL CONSISTS OF (8) EIGHT SENSORS IN A probe of which (4) four SENSORS MUST BE operable.

TABLE 4.3-7

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1. Containment Pressure	M	R
2. Reactor Coolant Outlet Temperature - T_{Hot} (Narrow Range)	M	R
3. Reactor Coolant Inlet Temperature -T_{Cold} (Wide Range)	M	R
4. Reactor Coolant Pressure - Wide Range	M	R
5. Pressurizer Water Level	M	R
6. Steam Generator Pressure	M	R
7. Steam Generator Water Level - Narrow Range	M	R
8. Steam Generator Water Level - Wide Range	M	R
9. Refueling Water Storage Tank Water Level	M	R
10. Auxiliary Feedwater Flow Rate (Each pump)	M	R
11. Reactor Coolant System Subcooling Margin Monitor	M	R
12. PORV Position/Flow Indicator	M	R
13. PORV Block Valve Position Indicator	M	R
14. Safety Valve Position/FLow Indicator	M	R
16. Containment Sump Water Level (Narrow Range)	M	R
16. Containment Water Level (Wide Range)	M	R
17. Incore Thermocouples	M	R
18. Reactor Vessel Level Monitoring System	M	R

ATTACHMENT 3

NO SIGNIFICANT HAZARDS CONSIDERATIONS

Introduction

The requested amendment to the St. Lucie Units 1 and 2 operating licenses adds plant instrumentation currently installed and operational to the Technical Specifications. This addition will provide a higher degree of control and operational readiness for these instrument channels by requiring operability and monthly surveillances of accuracy.

Evaluation

The three criteria for a determination of whether a no significant hazards consideration exists, from 10 CFR 50.92(c), are:

1) Involve a significant increase in the probability or consequences of an accident previously evaluated, or

2) create the possibility of a new or different kind of accident from any accident previously evaluated; or

3) involve a significant reduction in a margin of safety.

A. With respect to the first of the above criteria, the requested license amendment does not increase the probability or consequences of accidents previously analyzed. The plant hardware and normal operating conditions are not affected by the proposed change since this equipment was previously installed and operating. Addition of monthly surveillances on these instrument channels will not involve any significant increase in the probability or consequences of an accident previously evaluated but will assure accurate output from these instrument channels.

B. With respect to the second criteria, plant hardware and basic plant operation are not affected by the proposed license amendment. Therefore, the possibility for a new or different type of accident is not created.

C. With respect to the third criteria, since the consequences of accidents are not increased and new or different types of accidents are not introduced by this amendment, all margins of safety will be maintained.

It is concluded from the above evaluation that this amendment involves no significant hazards considerations. Furthermore, the proposed amendment is similar to example (11) of the Commissions examples of amendments that are considered not likely to involve significant hazards considerations in that it is a change that constitutes an additional limitation, restriction or control not presently in the technical specifications.

STATE OF FLORIDA)
) ss.
COUNTY OF DADE)

J. W. Williams, Jr. being first duly sworn, deposes and says:

That he is a Group Vice President of Florida Power & Light Company, the Licensee herein;

That he has executed the foregoing document; that the statements made in this document are true and correct to the best of his knowledge, information, and belief, and that he is authorized to execute the document on behalf of said Licensee.

J. W. Williams, Jr.

Subscribed and sworn to before me this
14th day of July, 1985.

Lourdes Jordan
NOTARY PUBLIC, in and for the County of Dade, State of Florida

My Commission expires: Dec 8, 1988