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Regulatory Docket File

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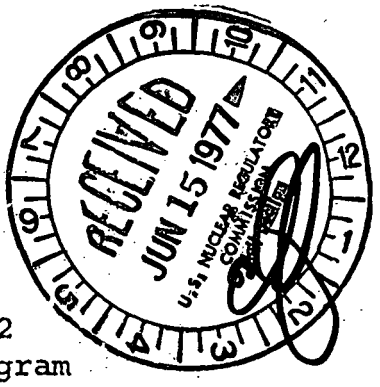

Mr. Donald K. Davis, Acting Chief
Operating Reactors - Branch 2
Division of Operating Reactors
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Dresden Station Units 2 and 3
Quad-Cities Station Units 1 and 2
Mark I Containment Long Term Program
Inspection Plan
NRC Docket Nos. 50-237/249/254/265

References (a): D. L. Ziemann letter to R. L. Bolger, dated January 8, 1976. (Enclosure 1 - Question SEB-5, SEB-30)

(b): Dresden Units 2 and 3 Plant Unique Analysis, Nutech Report COM-01-040, dated August, 1976 and COM-01-040, Rev. 1, dated August, 1976.

(c): G. A. Abrell letter to D. L. Ziemann, dated December 9, 1976. (Attachment: Nutech Report COM-01-066 dated December, 1976)

(d): Quad-Cities Units 1 and 2 Plant Unique Analysis, Nutech Report COM-01-039 dated August, 1976.

(e): Suppression Chamber Support Column - Shell Connection Capacity Evaluation for Quad-Cities Units 1 and 2, Nutech Report COM-01-021, Rev. 2, dated August, 1976.

Dear Mr. Davis:

A program of inspection has been defined for the critical Mark I Containment components at Dresden and Quad-Cities Stations. This program will complete Commonwealth Edison's response to Reference (a) by answering questions SEB-5 and SEB-30. The inspection plan addresses components identified as critical in the Mark I Short Term Program. These components have been identified as Category I, and include:

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1. Torus Support Columns

2. Torus Support Column to Shell Connections

3. Torus Support Column to Base Mat Connections

4. Piping Attached to the Torus

In addition, the inspection plan addresses those components for which detailed plant unique analyses were not performed as part of the Short Term Program, but which are recognized as critical to the continued safe operation of the containments. These components have been identified as Category II, and include:

1. Suppression Chamber - Shell/Reinforcing Ring Connection

2. Vent Header

3. Vent Header Support System

4. Vent Downcomers

5. Catwalk/Catwalk Supports

6. Miscellaneous Internals (e.g., Containment Spray, Monorail Supports, etc.)

The inspection of the Category I components was completed as part of the Short Term Program. The results of these inspections are documented in References (b), (c), (d), and (e). The final revision to Reference (e) is currently being prepared, and will be available for transmittal under separate cover in early June, 1977. It is noteworthy that the Category I components were also inspected on a sample basis by the NRC Regional inspector.

The inspection of the Category II components will be performed during the unit refueling outages in 1977 and 1978. More specifically, the schedule for these inspections is as follows:

Quad-Cities Unit 1 - April, 1977
Dresden Unit 2 - October, 1977
Quad-Cities Unit 2 - February, 1978
Dresden Unit 3 - April, 1978

These Category II inspections must be scheduled during unit refueling outages because the components in this group are all internal to the torus. Furthermore, access to the regions of interest require that the torus water level be lowered which is only done during scheduled refueling outages, as opposed to short maintenance outages.

The Category I inspections were both visual and mechanical, the latter including measurement of weld sizes and lengths. The Category II inspections also include both visual and mechanical observations. Inasmuch as the torus internal coating is relatively thick and the environmental conditions for inspection are more severe than for external inspections, the visual inspection will be limited to gross deviations, i.e., lack of weld where specified, excessive weld buildup, etc. Measurements will be made of critical welds on the reinforcing ring to shell connection and the vent header support system. In addition, critical gaps and component sizes will be examined to confirm design acceptability, e.g., vent header support - clevis plate dimensions, vent downcomer to bellows clearances, etc.

The results of these inspections will be documented after they have been reviewed by the Station Nuclear Engineering Department. Inasmuch as these inspections are not being performed to satisfy ASME Code requirements, NDE qualifications for personnel in accordance with Section V of the ASME Code are not applicable. However, all inspections will be performed by individuals trained specifically for this work.

One (1) signed original and 39 copies of this letter are being transmitted for your use.

Very truly yours,

M. S. Turbak

M. S. Turbak
Nuclear Licensing Administrator
Boiling Water Reactors