



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

January 5, 1977

Docket Nos. 50-237, 50-249,  
50-254, 50-265

Commonwealth Edison Company  
ATTN: Mr. R. L. Bolger  
Assistant Vice President  
Post Office Box 767  
Chicago, Illinois 60690

Gentlemen:

RE: DRESDEN UNIT NOS. 2 AND 3, QUAD CITIES UNIT NOS. 1 AND 2

We have noted a potential deficiency in the method being used to confirm valve operability during periodic testing of Boiling Water Reactor (BWR) safety-relief valves. This deficiency concerns the use of the safety-relief valve temperature indication as a positive method of confirmation that a safety-relief valve is open when manually actuated during surveillance testing.

We have found that an increased temperature indication may be obtained at the safety-relief valve exit with the safety-relief valve closed. This indicated temperature increase is the result of steam vented through the valve actuation mechanism during the surveillance test. In view of this finding, we have concluded that a temperature increase at the valve exit, by itself does not provide a positive means of verification that the safety-relief valve has opened.

We have reviewed your Technical Specifications and have determined that they do not provide adequate surveillance requirements to assure safety-relief valve operability. Therefore, we require that you eliminate this potential deficiency in your surveillance testing program. You should propose a change to your technical specifications that provides a positive means of verification that the safety-relief valve opens when tested as part of the Automatic Depressurization System. An example of an acceptable method would involve the manual actuation of each individual safety-relief valve at one or more specified pressures and observing a specified minimum compensating turbine bypass or throttle valve closure or change in measured steam flow rate.

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Gentlemen:

Your letter of December 13, 1976, noted that no interval for high pressure coolant injection (HPCI) system flow testing is specified in the Technical Specifications for Dresden Unit Nos. 2 and 3. In addition, we note that no interval is specified for flow rate tests of core spray and low pressure cooling systems. The omission of these test intervals is an oversight which should be corrected. We therefore request that you submit a request for an amendment to specify the test frequencies. The request should be submitted within 60 days of the date of this letter.

Sincerely,

Dennis L. Ziemann, Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

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