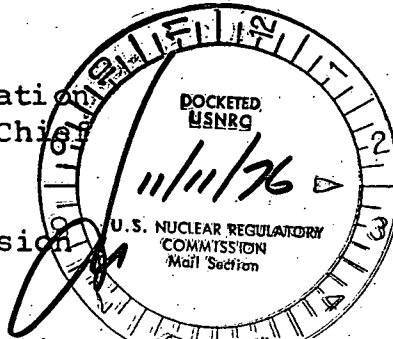




**Commonwealth Edison**  
 One First National Plaza, Chicago, Illinois  
 Address Reply to Post Office Box 767  
 Chicago, Illinois 60690

**REGULATORY DOCKET FILE COPY**  
 November 10, 1976

Director Nuclear Reactor Regulation  
 Attn: Mr. Dennis L. Ziemann, Chief  
 Operating Reactors - Branch 2  
 Division of Operating Reactors  
 U.S. Nuclear Regulatory Commission  
 Washington, D.C. 20555



**Subject:** Dresden Station Units 2<sup>0</sup> and 3  
 Quad-Cities Station Units 1 and 2  
 Effects of Core Flow on Emergency  
 Core Cooling System (ECCS) on Loss  
 of Coolant Accident (LOCA) Analysis  
NRC Docket Nos. 50-237/249 and 50-254/265

**References (a):** D. L. Ziemann Letter to R. L. Bolger dated  
 October 13, 1976 - NRC Docket Nos.  
 50-237/249 and 50-254/265.

**(b):** A. J. Levine (GE) Letter to Z. R. Rosztoczy  
 (NRC) dated August 24, 1976.

Dear Mr. Ziemann:

Commonwealth Edison has reviewed Reference (b) in accordance with Reference (a) and finds that Reference (b) has no impact on the Dresden Units 2 and 3 or Quad-Cities Units 1 and 2 reactors. These vessels are 251 inches ID and fall into the category of "all other BWR's" on Table C.4.5.7-1. This has been confirmed by General Electric Company.

Commonwealth Edison imposed a five percent reduction in the maximum average planar linear heat generation rate (MAPLHGR) below 85 percent core flow when informed by General Electric Company of the potential problem of elevated peak clad temperature (PCT) calculated for a LOCA at reduced core flow. This restriction was removed when informed by General Electric Company that it did not apply to our units.

One (1) signed original and 59 copies are provided for your use.

Very truly yours,

11504

G. A. Abrell  
 Nuclear Licensing Administrator  
 Boiling Water Reactors