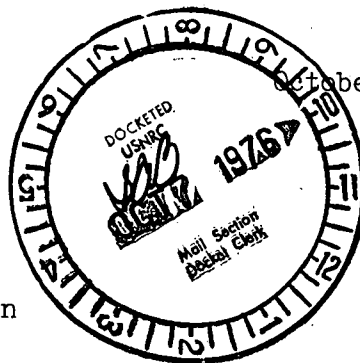




Commonwealth Edison
 One First National Plaza, Chicago, Illinois
 Address Reply to: Post Office Box 767
 Chicago, Illinois 60690

Regulatory

File Cy.



October 1, 1976



Mr. Dennis L. Ziemann, Chief
 Operating Reactors - Branch 2
 Division of Operating Reactors
 U.S. Nuclear Regulatory Commission
 Washington, D.C. 20555

Subject: Mark I Containment Short Term Program
 Response to Regulatory Questions
 Dresden Station Units 2 and 3 and Quad
 Cities Station Units 1 and 2; Docket
 No.'s 50-236, 50-249, 50-254, 50-265

Dear Mr. Ziemann:

In response to requests directed to the Mark I Utilities through General Electric (the Mark I Containment Evaluation Program Manager), Commonwealth Edison has:

1. reviewed the information in Table 2-1a of Addendum 2 and Table 6-1 of Addendum 3 of the Mark I Containment Evaluation - Short Term Program Final Report;
2. evaluated the effect on the plant unique containment analysis for maximum torus water level, and the loads associated with this condition; and
3. conducted a sensitivity study to determine the effect of not taking credit for the reduction in vent system loads that result for Quad Cities Units 1 & 2 from the use of Curve 3-9 of Addendum 3 of the Mark I Containment Evaluation - Short Term Program Final Report, NEDC-20909-P.

It has been determined that the information contained in the tables from Addenda 2 and 3, as referenced above, require modification. Commonwealth Edison has reviewed and confirmed values for all the parameters specific to Dresden Units 2 and 3, and Quad Cities Units 1 and 2 which have been provided verbally to the NRC Staff by General Electric. The corrected tables are provided as a part of Appendix A to the Supplements to the Dresden and Quad Cities plant unique evaluation reports which will be transmitted under separate cover. It has been determined by reanalysis that the effect of the revised plant specific parameters is insignificant. The results of this reanalysis are documented in the Supplements to the Dresden and Quad Cities plant unique evaluation reports.

It has also been shown that the commitment contained in our letter of February 6, 1976 to Mr. B.C. Rusche to operate as closely as possible to the minimum technical specification limiting condition of operation has been followed. At Quad Cities Station, both Units 1 and 2 have operated within 2" of the technical specification minimum (4" total range minimum to maximum) 90% of the time. At Dresden Station, both Units 2 and 3 have operated within 2" of the technical specification

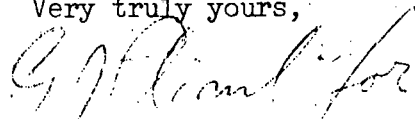
minimum (4" total range minimum to maximum) 70% of the time. The Dresden and Quad Cities plant unique evaluations have been re-examined to determine the impact of operation at water levels greater than the minimum technical specification limit. As documented in the Supplements to the Dresden and Quad Cities plant unique evaluation reports, the Mark I Containment Short Term Program Criteria are not exceeded at any water level within the technical specification water level control band.

In addition, the results of the sensitivity study performed for Quad Cities Units 1 & 2, to establish the impact of not taking credit for the reduction in vent system loads that result from the use of Curve 3-9 in Addendum 3 to the Mark I Short Term Program Final Report, indicate that the torus support structure and attached piping remain well within the Short Term Program criteria. The results of this study are summarized in Nutech letter COM-01-052, dated October 1, 1976.

This letter documents the results of plant unique assessments of revised plant unique geometry factors and the associated plant unique load factors (which factors were originally documented in Addenda 2 and 3 of the Mark I Containment Evaluation - Short Term Program Final Report) for Dresden and Quad Cities Stations. The revised factors are submitted as an Appendix to the Supplements to the Dresden and Quad Cities plant unique evaluation reports. This letter also documents the results of plant unique assessments of plant operation at torus water levels up to the maximum allowed by the technical specification. These additional analyses, when compared to the plant unique reports previously submitted, indicate that corrections made to the plant unique parameters have an insignificant effect on computed strength ratios and piping stresses, and that assumed operation at the maximum allowed torus water level causes a nominal increase in computed strength ratios and piping stresses. In addition, this letter documents the results of the sensitivity study for Quad Cities Units 1 and 2 which confirms that the use of Curve 3-9 of NEDC-20909-P, Addendum 3, has a nominal effect on computed piping stresses and strength ratios. In no case do the analytical results violate the Short Term Program criteria.

One (1) signed original and 39 copies of this letter are being transmitted for your use. Forty (40) copies of Nutech Report COM-01-050, "Quad Cities Units 1 and 2, Supplement to Short Term Program Plant Unique Torus Support and Attached Piping Analyses", 40 copies of Nutech Report COM-01-051, Dresden Units 2 and 3, Supplement to Short Term Program Plant Unique Torus Support and Attached Piping Analyses", and 40 copies of Nutech letter COM-01-052, dated October 1, 1976 (Mr. G.E. Edwards, Nutech, to Mr. D.P. Galle, ComEd), will be forwarded under separate cover by Nutech. This letter and the referenced reports should be included on the Dresden and Quad Cities dockets.

Very truly yours,



G.A. Abrell
Nuclear Licensing Administrator
Boiling Water Reactors

LOD/tlf
Attachments