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ACCESSION NBR: 8411260257 DOC. DATE: 84/11/20 NOTARIZED: NO  
 FACIL: 50-389 St. Lucie Plant, Unit 2, Florida Power & Light Co.  
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 RECIP. NAME: EISENHUT, D.G. RECIPIENT AFFILIATION: Division of Licensing

DOCKET #  
05000389

SUBJECT: Forwards status of resolution of license conditions covering inservice insp program, axial growth & fire protection. Deletion of completed conditions in future license amend requested.

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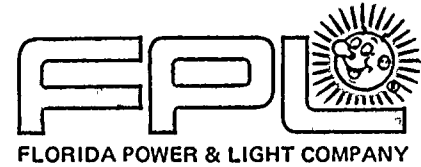
TO: DIRECTOR, FBI  
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SUBJECT: [Illegible]

Reference is made to the report of [Illegible] dated [Illegible] regarding [Illegible].

It is noted that [Illegible] is currently [Illegible] and [Illegible].

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November 20, 1984

L-84-333

Office of Nuclear Reactor Regulation  
Attention: Mr. Darrell G. Eisenhut, Director  
Division of Licensing  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Mr. Eisenhut:

Re: St. Lucie Unit No. 2  
Docket No. 50-389  
Resolution of License Conditions

The attachment to this letter provides a status of the St. Lucie 2 license conditions.

It is requested that those license conditions, which have been completed to NRC's satisfaction be deleted in a future license amendment.

Very truly yours,

A handwritten signature in cursive script, appearing to read "J. Williams, Jr.", written in dark ink.

J. W. Williams, Jr.  
Group Vice President  
Nuclear Energy

JWW/RJS/cab

Attachment

cc: James P. O'Reilly, Region II

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**ST. LUCIE UNIT NO. 2  
OPERATING LICENSE NPF-16  
STATUS OF LICENSE CONDITIONS**

**2.C.4. Environmental Qualification of Mechanical and Electrical Equipment (Section 3.11, SSER 4)**

Prior to startup following the first refueling outage but not later than March, 1985 the licensee shall environmentally qualify all electrical equipment as required by 10 CFR 50.49.

**Status:** All equipment required to be qualified by 10 CFR 50.49, has been qualified. This license condition has been satisfied.

**2.C.5 Axial Growth (Section 4.2.3.1(g), SSER 3)**

Prior to startup following the first refueling outage, the licensees shall provide an analysis and/or make hardware modifications to assure that the shoulder gap clearance between fuel rods and fuel assembly end fittings is adequate.

**Status:** The results of the shoulder gap inspection and an analysis to assure that the shoulder gap clearance is adequate for all future cycles were submitted to NRC on November 8, 1984, L-84-314. This license condition has been satisfied.

**2.C.6. Inservice Inspection Program for Class 1, 2 and 3 Components (Section 5.2.4, 6.6, SER)**

Within six (6) months from the date of this license, the licensees shall submit a revised inservice inspection program for Class 1, 2 and 3 components for NRC approval.

**Status:** The revised inservice inspection program for Class 1, 2 and 3 components was submitted for NRC approval on October 6, 1983, L-83-510. This license condition has been satisfied.

**2.C.7 Natural Circulation Cooldown and Boron Mixing Test (Section 5.4.3, SSER 4)**

Prior to completing the startup test program, the licensees shall either a) provide a report of the San Onofre Unit 2 test justifying that the test data is applicable to St. Lucie 2 assuring adequate boron mixing during natural circulation cooldown or b) perform the test to demonstrate adequate boron mixing during natural circulation cooldown.

**Status:** A report of the San Onofre Unit 2 test was provided to NRC on August 3, 1983, L-83-437, and a detailed report was submitted on March 13, 1984, L-84-68. This license condition has been satisfied.

2.C.8 Continuous Containment Purge System (Section 6.2.4, SSER 3)

Prior to exceeding 5% of rated thermal power, the licensees will make the necessary modifications to assure the operability of the Continuous Purge System in the event of a loss of coolant accident.

Prior to startup following the first refueling outage, the licensees shall install testing capability for the 8 inch purge valves which would allow for testing to the Standard Technical Specifications requirements of every 92 days.

Status: The necessary modifications to assure operability of the Continuous Purge System were completed prior to exceeding 5% of rated thermal power.

The Continuous Containment Purge System was modified during the first refueling outage to allow for testing to the Standard Technical Specifications requirements.

This license condition has been satisfied.

2.C.9. Barrier for High Energy Equipment (8.4.1, SSER 3)

Prior to startup following the first refueling outage, the licensees shall have installed a barrier around the transformer in the cable spreading area that is acceptable to the NRC. Prior to installation, the licensees shall submit for NRC review and approval the barrier design to be used and justification for its acceptability.

Status: The barrier design was submitted to NRC on February 27, 1984, L-84-44, and additional information on June 27, 1984, L-84-160. NRC approved the design on July 17, 1984. The barrier was installed during the first refueling outage. This license condition has been satisfied.

2.C.10. Non-Safety Loads on Emergency Power Sources (Section 8.4.2, SER, SSER 3)

Prior to startup following the first refueling outage, the licensees shall implement the design modification to disconnect four-kilovolt loads on detection of a safety injection signal and provide two isolation devices in series for those non-safety electrical loads that are not disconnected by a safety injection signal or loss of offsite power.

Status: Modifications have been implemented to disconnect the 4KV loads on a safety injection signal and to provide two isolation devices in series for those non-safety electrical loads that are not disconnected by a safety injection signal or loss of offsite power. This license condition has been satisfied.



2.C.11. Containment Electrical Penetrations (Section 8.4.3, SSER 3)

Prior to startup following the first refueling outage, the licensees shall complete the design modifications to provide independent primary and backup fault protection for each electrical conductor penetrating containment.

Status: Modifications have been implemented to provide independent primary and backup fault protection for each electrical conductor penetrating containment, as required. This license condition has been satisfied.

2.C.12. Heavy Loads (Section 9.1.4, SSER 3)

Prior to startup following the first refueling outage, the licensees shall conform to the guidelines of Section 5.1.1 of NUREG-0612 and prior to thirty days of startup following the second refueling outage, the licensees shall have made commitments acceptable to the NRC regarding the guidelines of Section 5.1.2 through 5.1.6 of NUREG-0612.

Status: Conformance to the guidelines of Section 5.1.1 of NUREG-0612 was provided in letters dated August 6, 1981 (L-81-338), September 21, 1982 (L-82-409), and November 9, 1984 (L-84-327). This portion of the license condition has been satisfied.

Regarding the guidelines of Section 5.1.2 through 5.1.6 of NUREG-0612, prior to thirty days of startup following the second refueling outage, commitments acceptable to NRC will be made. This portion of the license condition remains open.

2.C.13 Fire Protection (Section 9.5.1.11(a) and (b), SSER 3)

The licensees shall implement the fire protection program on a schedule specified in Section 9.5.1.11(a) and (b) of Supplement No. 3 to the Safety Evaluation Report.

Status: The fire protection program was implemented as specified in Section 9.5.1.11 (a) and (b) of Supplement No. 3 to the Safety Evaluation Report. This license condition has been satisfied.

2.C.14 Emergency Diesel Generator Modifications (Section 9.5.4.1, SER)

Prior to startup following the first refueling outage, the licensees shall a) install and have fully operational the automatic prelube pump and b) relocate instruments and controls located on the diesel engine skid to the floor-mounted panel.

Status: The automatic prelube pump has been installed and is operable, and the instruments and controls have been relocated to the floor-mounted panel. This license condition has been satisfied.

2.C.15. Radioactive Waste Management (Section 11.1, 11.5, SER, SSER 3)

Within 14 months after core load, the licensees shall 1) implement the design modifications to automatically shut off the waste management condensate and boric acid condensate pumps prior to the level reaching the overflow nozzle of the Primary Water Storage Tank and b) implement the design modification to automatically isolate the Low Pressure Safety Injection pump discharge to the Refueling Water Tank upon receipt of a refueling water tank high water level alarm.

Prior to startup following the first refueling outage, FPL shall 1) install waste concentrator bottom tanks (s.i.c.) and, b) install a second continuous oxygen analyzer.

Status: Modifications have been made to automatically shut off the waste management condensate and boric acid condensate pumps prior to the level reaching the overflow nozzle of the Primary Water Storage Tank.

Modifications have been made to automatically isolate the Low Pressure Safety Injection pump discharge to the Refueling Water Tank upon receipt of a refueling water tank high water level alarm.

A waste concentrator bottoms tank and a second continuous oxygen analyzer have been installed.

This license condition has been satisfied.

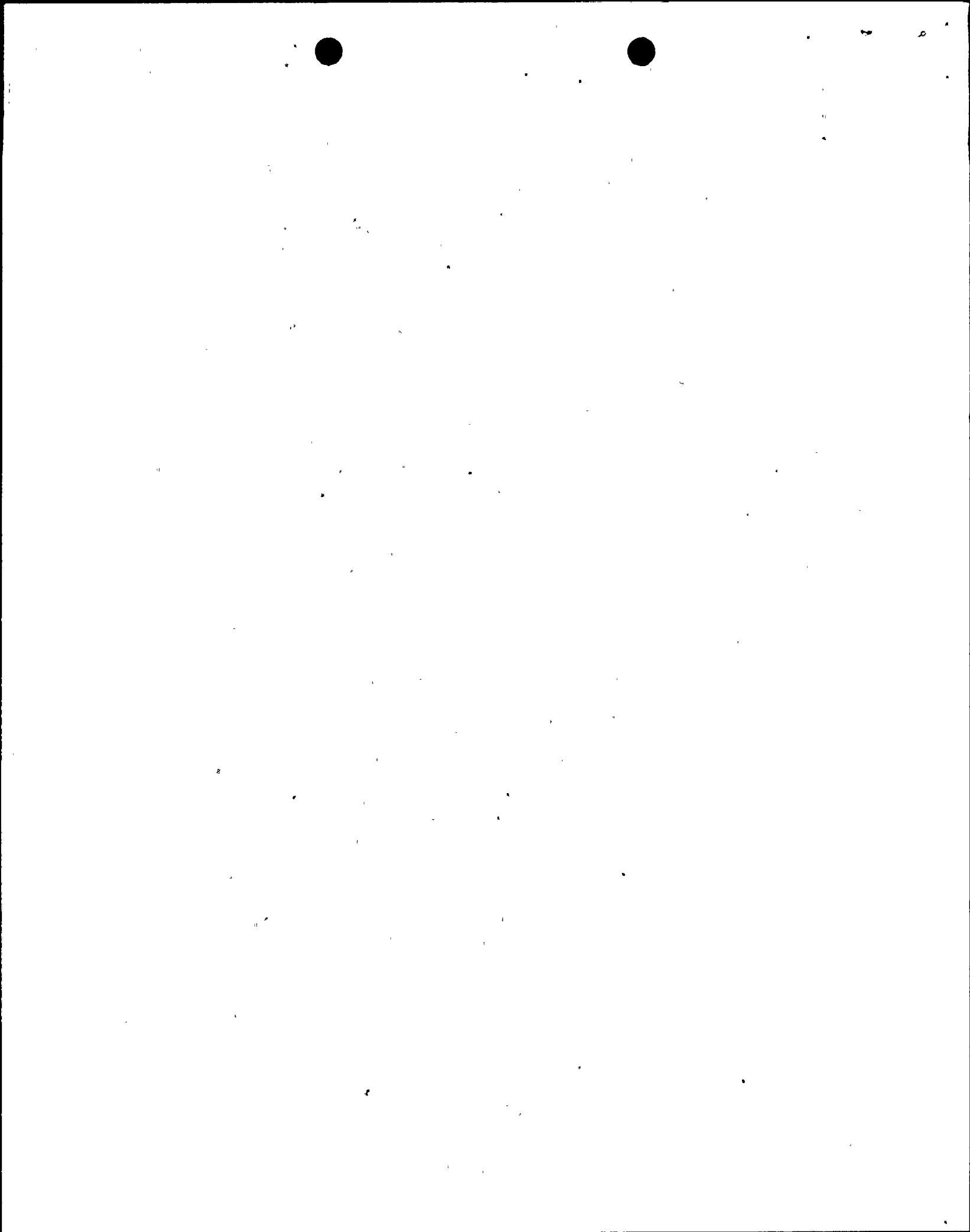
2.C.16. Initial Test Program (Section 14, SER)

The licensees shall conduct the post-fuel loading initial test program (set forth in Section 14 of the St. Lucie 2 Final Safety Analysis Report, as amended through Amendment 13 and FPL's letter L-83-207) without making any modifications to this program unless such modifications are in accordance with the provisions of 10 CFR Section 50.59. In addition, the licensees shall not make any major modifications to this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- a. Elimination of any test identified as essential in Section 14 of the Final Safety Analysis Report, as amended through Amendment 13 and FPL's letter L-83-207;
- b. Modification of test objectives, methods, or acceptance criteria for any test identified as essential in Section 14 of the Final Safety Analysis Report, as amended through Amendment 13 and FPL's letter L-83-207;
- c. Failure to complete any tests included in the described program (planned or scheduled) for power levels up to the authorized power level.

Status: The Initial Test Program was completed and reported in letter L-83-542, dated November 1, 1983. This license condition has been satisfied.





2.C.17 NUREG-0737 Conditions (Section 22, SER, Section 13.3, SSER 3)

a. Emergency Response Capability (I.C.9, I.D.1, I. D.2, 7.5.4, 13.3.2.8, SSER 4)

- 1) The licensee shall implement the requirements of Generic Letter No. 82-33 on the following schedule:
  - a) Prior to startup following the first refueling outage, the Safety Parameter Display System (SPDS) shall be operable, including training of operators.
  - b) By September 30, 1983 the licensee shall submit for NRC review and approval a detailed control room design review summary report.
  - c) By November 30, 1983 the licensee shall submit for NRC review and approval a Regulatory Guide 1.97 Evaluation Report describing how Regulatory Guide 1.97 has been met and justifications for any deviations.
  - d) By November 1, 1983 the licensee shall submit for NRC review and approval plant specific Emergency Operating Procedures descriptions. By July 1, 1984, the licensee shall implement the upgraded Emergency Operating Procedures including SPDS and control room upgrades.
  - e) By October 1983, the permanent Emergency Operating Facility shall be operational.

Status:

- a) The SPDS has been installed and is operable, and operators have been trained on the system.

This license condition has been satisfied.

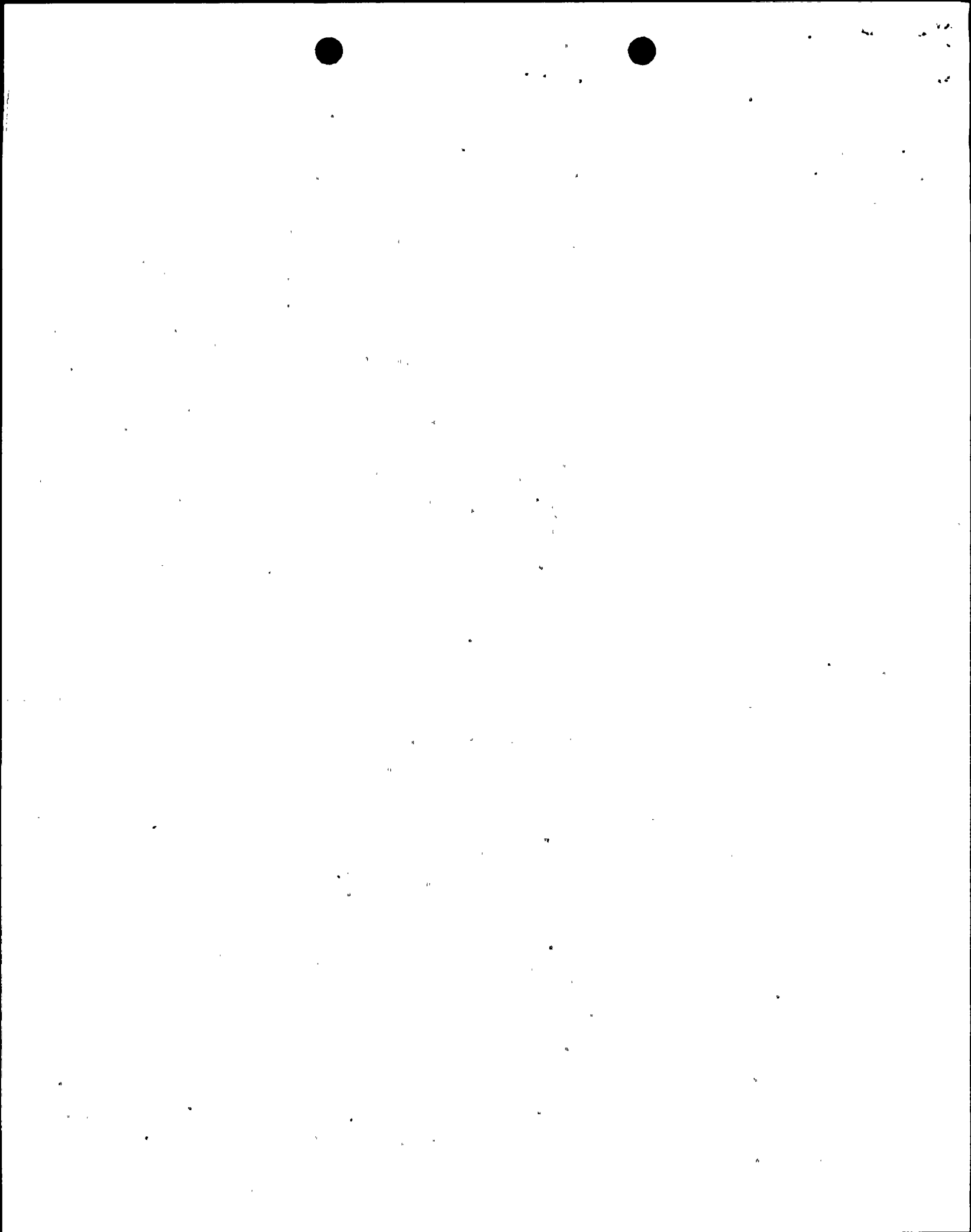
- b) The Detailed Control Room Design Review (DCRDR) was submitted to NRC on September 30, 1983 (L-83-504). This license condition has been satisfied.

- c) The Regulatory Guide 1.97 Evaluation was submitted to NRC on November 30, 1983 (L-83-573). This license condition has been satisfied.

- d) Plant Specific Emergency Operating Procedures descriptions were submitted to NRC on October 26, 1983 (L-83-536). This portion of the license condition has been satisfied.

Implementation of the upgraded Emergency Operating Procedures is scheduled for July 1, 1985 per NRC Confirming Order dated June 14, 1984. This portion of the license condition remains open.

- e) The Emergency Operations Facility was completed and



declared operational on September 1, 1983. See FPL letter L-83-578 dated December 7, 1983. This license condition has been satisfied.

- 2) The licensees shall maintain interim emergency support facilities (Technical Support Center, Operations Support Center and the Emergency Operations Facility) until the upgraded facilities are completed.

**Status:** Interim emergency support facilities were maintained while the upgraded facilities were being completed. This license condition has been satisfied.

- b. Control Room Design Review (I.D.1, Appendix E, Also Part A of Appendix C, SSER 1, SSER 3)

Prior to exceeding five (5) percent of rated thermal power, the licensees shall complete correction of the human engineering discrepancies as noted in Appendix E of this license.

Prior to startup following the first refueling outage, the licensees shall rearrange the instruments described in Appendix F.

**Status:** The human engineering discrepancies as noted in Appendix E were corrected prior to exceeding five percent of rated thermal power. The instruments described in Appendix F were rearranged as required during the first refueling outage. This license condition has been satisfied.

- c. Reactor Coolant System Vents (II.B.1, SSER 2)

Prior to exceeding five (5) percent of rated thermal power, FPL shall have the reactor coolant system vents installed and operational.

**Status:** The reactor coolant system vents were installed and operational prior to exceeding five percent of rated thermal power. This license condition has been satisfied.

- d. Post Accident Sampling System (II.B.3, SSER 3)

Prior to exceeding initial criticality, FPL shall have installed and operational the Post Accident Sampling System.

Prior to startup following the first refueling outage, the licensees shall submit for NRC approval a revised core damage assessment procedure which incorporates, as a minimum, hydrogen levels, reactor coolant system pressure, core exit thermocouple temperatures and containment radiation levels in addition to radionuclide data.

Status: The Post Accident Sampling System was installed and operational prior to exceeding initial criticality.

A revised core damage assessment procedure was forwarded to NRC for approval on November 9, 1984 (L-84-320).

This license condition has been satisfied.

e. In-Containment High Range Radiation Monitors (II.F.1(2c), SSER 3)

Prior to exceeding five (5) percent of rated thermal power, FPL shall have the in-containment high range radiation monitors installed and operational.

Status: The in-containment high range radiation monitors were installed and operational prior to exceeding five percent of rated thermal power. This license condition has been satisfied.

f. Inadequate Core Cooling Instrumentation (II.F.2, SSER 1)

The licensees shall have:

- 1) The final design core exit thermocouple (CET) system installed with displays in the Qualified Safety Parameter Display System (QSPDS) cabinets, by initial criticality.
- 2) The instrumentation necessary to monitor and display subcooling margin installed in the QSPDS cabinets and operable by initial criticality.
- 3) The heated junction thermocouples (HJTC) installed in the QSPDS cabinets and operable by June 1983.
- 4) The final Inadequate Core Cooling System checkout and test report completed and submitted by January 15, 1984.

Status: 1) The CET system with displays in the QSPDS was installed by initial criticality.

2) The SMM was installed and operational by initial criticality.

3) The HJTC were installed and operable by June 1983.

4) The final ICCS checkout was completed in June 1983, and the test report was submitted to NRC on February 3, 1984, L-84-27.

The license condition has been satisfied.

18. Reactor Trip Breakers Post-Trip Review Procedures (Section 7.2.3.1, SSER 4)

Within 60 days of issuance of this amendment, the licensee shall provide the upgraded post-trip review procedures for NRC staff review.

Status: The post-trip review procedures were submitted for NRC staff review on August 11, 1983, L-83-444. This license condition has been satisfied.