

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 68 License No. DPR-67

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, (the licensee) dated May 21, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 68, are hereby incorporated in the license: The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James R. Miller, Chief Operating Reactors Branch #3

Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 5, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 68

FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Remove and replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are provided to maintain document completeness.

Remove	Insert
V X B 3/4 4-15	V X 3/4 4-61 B 3/4 4-15

	CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS	
SECTION		<u>PAGE</u>
3/4.6 C	ONTA TAIMENT SVOTEMO	2
	ONTAINMENT SYSTEMS	
3/4.6.1	CONTAINMENT VESSEL	•
	Containment Vessel Integrity	•
	Containment Leakage	•
	Containment Air Locks	•
	Internal Pressure	3/4 6-12
	Air Temperature	3/4 6-13
	Containment Vessel Structural Integrity	3/4 6-14
3/4.6.2	DEPRESSURIZATION AND COOLING SYSTEMS	3/4 6-15
	Containment Spray System	3/4 6-15
	Spray Additive System	3/4 6-16
	Containment Cooling System	3/4 6-17
3/4.6.3	CONTAINMENT ISOLATION VALVES	3/4 6-18
3/4.6.4	COMBUSTIBLE GAS CONTROL:	3/4 6-23
٠	Hydrogen Analyzers	3/4 6-23
	Electric Hydrogen Recombiners - W	3/4 6-24
3/4.6.5	VACUUM RELIEF VALVES	
3/4.6.6	SECONDARY CONTAINMENT	
	Shield Building Ventilation System	-
	Shield Building Integrity	
	Shield Building Structural Integrity	•
3/4.7 PL	ANT SYSTEMS	
3/4.7.1	TURBINE CYCLE	3/4 7-1
	Safety Valves	
	Auxiliary Feedwater System	
	Condensate Storage Tank	
	Activity	
	Main Steam Line Isolation Valves	
	Secondary Water Chemistry	
CT LUATE		3/4 /-10 ent No.27

LIMITING CONDITION FOR OPERATION AND SURVEILLANCE REQUIREMENTS		 	
SECTION .	Ē	PAGE	
3/4.4.4 PRESSURIZER	3/4	4-4	
3/4.4.5 STEAM GENERATORS	3/4	4-5	
3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGE	3/4	4-12	
Leakage Detection Systems	3/4 3/4	4-12 4-14	
3/4.4.7 CHEMISTRY	3/4	4-15	
3/4.4.8 SPECIFIC ACTIVITY	3/4	4-17	
3/4.4.9 PRESSURE/TEMPERATURE LIMITS	3/4	4-21	
Reactor Coolant SystemPressurizer	3/4	4-25	
3/4.4.10 STRUCTURAL INTEGRITY	3/4	4-26	
Safety Class 1 Components	3/4	4-37	
3/4.4.11 CORE BARREL MOVEMENT.	3/4	4-56	
3/4.4.12 PORV BLOCK VALVES	3/4	4-58	
3/4.4.13 POWER OPERATED RELIEF VALVES	3/4	4-59	
3/4.4.14 REACTOR COOLANT PUMP - STARTING	3/4	4-60	
3/4.4.15 REACTOR COOLANT SYSTEM VENTS	3/4	4-61	
3/4.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)			
3/4.5.1 SAFETY INJECTION TANKS			
3/4.5.2 ECCS SUBSYSTEMS - Tavg > 325°F	3/4	5-3	
3/4.5.3 ECCS SUBSYSTEMS - Tavg < 325°F			
3/4.5.4 REFUELING WATER TANK	3/4	5-8	

BASES	<u> </u>			
SECTION	•		PAGE	<u> </u>
3/4.4 RE	ACTOR COOLANT SYSTEM			
3/4.4.1	REACTOR COOLANT LOOPS AND COOLANT CIRCULATION	В	3/4	4-1
3/4.4.2	and 3/4.4.3 SAFETY VALVES	В	3/4	4-1
3/4.4.4	PRESSURIZER	В	3/4	4-2
3/4.4.5	STEAM GENERATORS	В	3/4	4-2
3/4.4.6	REACTOR COOLANT SYSTEM LEAKAGE	В	3/4	4-4
3/4.4.7	CHEMISTRY	В	3/4	4-4
3/4.4.8	SPECIFIC ACTIVITY	В	3/4	4-5
3/4.4.9	PRESSURE/TEMPERATURE LIMITS	В	3/4	4-6
3/4.4.10	STRUCTURAL INTEGRITY	В	3/4	4-12
3/4.4.11	CORE BARREL MOVEMENT	В	3/4	4-13
3/4.4.12	PORV BLOCK VALVES	В	3/4	4-14
3/4.4.13	POWER OPERATED RELIEF VALVES and 3/4.4.14 REACTOR COOLANT PUMP - STARTING	В	3/4	4-15
3/4.4.15	REACTOR COOLANT SYSTEM VENTS	В	3/4	4-15
3/4.5 EM	HERGENCY CORE COOLING SYSTEMS (ECCS)			
3/4.5.1	SAFETY INJECTION TANKS	В	3/4	5-1
3/4.5.2 a	and 3/4.5.3 ECCS SUBSYSTEMS	В	3/4	5-1
3/4.5.4	REFUELING WATER STORAGE TANK (RWST)	В	3/4	5-2
3/4.6 CC	NTAINMENT SYSTEMS			
3/4.6.1	CONTAINMENT VESSEL	В	3/4	6-1
3/4.6.2	DEPRESSURIZATION AND COOLING SYSTEMS	В	3/4	6-2
3/4.6.3	CONTAINMENT ISOLATION VALVES	В	3/4	6-3
3/4.6.4	COMBUSTIBLE GAS CONTROL	В	3/4	6-3
3/4.6.5	VACUUM RELIEF VALVES	В	3/4	6-4
3/4.6.6	SECONDARY CONTAINMENT	В	3/4	6-4

BASES		
SECTION		<u>PAGE</u>
3/4.7 PL	ANT SYSTEMS	
3/4.7.1	TURBINE CYCLE	B 3/4 7-1
3/4.7.2	STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION	B 3/4 7-3
3/4.7.3	COMPONENT COOLING WATER SYSTEM	B 3/4 7-4
3/4.7.4	INTAKE COOLING WATER SYSTEM	B 3/4 7-4
3/4.7.5	ULTIMATE HEAT SINK	B 3/4 7-4
3/4.7.6	FLOOD PROTECTION	B 3/4 7-4
3/4.7.7	CONTROL ROOM EMERGENCY VENTILATION SYSTEM	B 3/4 7-4
3/4.7.8	ECCS AREA VENTILATION SYSTEM	B 3/4 7-5
3/4.7.9	SEALED SOURCE CONTAMINATION	B 3/4 7-5
3/4.7.10	SNUBBERS	B 3/4 7-5
3/4.7.11	FIRE SUPPRESSION SYSTEMS	B 3/4 7-7
3/4.7.12	PENETRATION FIRE BARRIERS	B 3/4 7-7
3/4.8 EL	ECTRICAL POWER SYSTEMS	B 3/4 8-1
3/4.9 RE	FUELING OPERATIONS	
3/4.9.1	BORON CONCENTRATION	B 3/4 9-1
3/4.9.2	INSTRUMENTATION	B 3/4 9-1
3/4.9.3	DECAY TIME	8 3/4 9-1
3/4.9.4	CONTAINMENT PENETRATIONS	B 3/4 9-1
3/4.9.5	COMMUNICATIONS	B 3/4 9-1
3/4.9.6	MANIPULATOR CRANE OPERABILITY	B 3/4 9-1
3/4.9.7	CRANE TRAVEL - SPENT FUEL STORAGE BUILDING	B 3/4 9-2
3/4.9.8	SHUTDOWN COOLING AND COOLANT CIRCULATION	B 3/4 9-2

REACTOR COOLANT SYSTEM

3/4.4.15 REACTOR COOLANT SYSTEM VENTS

LIMITING CONDITION FOR OPERATION

- 3.4.15 At least one Reactor Coolant System vent path consisting of two vent valves and one block valve powered from emergency buses shall be OPERABLE and closed at each of the following locations:
 - a. Pressurizer steam space, and
 - b. Reactor vessel head.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION: `

- a. With one of the above Reactor Coolant System vent paths inoperable, STARTUP and/or POWER OPERATION may continue provided the inoperable vent path is maintained closed with power removed from the valve actuator of all the vent valves and block valves in the inoperable vent path; restore the inoperable vent path to OPERABLE status within 30 days, or be in HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With both Reactor Coolant System vent paths inoperable, maintain the inoperable vent paths closed with power removed from the valve actuators of all the vent valves and block valves in the inoperable vent paths, and restore at least one of the vent paths to OPERABLE status within 72 hours or beein HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

- 4.4.15 Each Reactor Coolant System vent path shall be demonstrated OPERABLE at least once per 18 months by:
 - 1. Verifying all manual isolation valves in each vent path are locked in the open position.
 - 2. Cycling each vent valve through at least one complete cycle of full travel from the control room.
 - 3. Verifying flow through the Reactor Coolant System vent paths during venting.

BASES

3/4.4.13 POWER OPERATED RELIEF VALVES and 3/4.4.14 REACTOR COOLANT PUMP - STARTING

The low temperature reactor coolant system overpressure mitigating system is provided to prevent RCS overpressurization above the 10 CFR 50, Appendix G, operating limit curves (Figure 3.4-2b or 3.4-2c, as applicable) at RCS temperatures below 275°F. The RCS overpressurization system is based on the use of the pressurizer power operated relief valves (I-V-1402 and I-V-1404) for the design basis mass injection transient, and the formation of a 60% pressurizer bubble by volume for the design basis energy addition transient. For the case when no pressurizer steam bubble is formed, protection against the design basis energy addition transient is derived by limiting the secondary-to-primary temperature differential below 50°F. The operability of the RCS overpressurization protection system will only be required during periods of heatup and cooldown below RCS temperatures below 275°F and periods of cold shutdown when the RCS has pressure boundary integrity.

3/4.4.15 REACTOR COOLANT SYSTEM VENTS

Reactor Coolant System vents are provided to exhaust noncondensible gases and/or steam from the primary system that could inhibit natural circulation core cooling. The OPERABILITY of at least one Reactor Coolant System vent path from the reactor vessel head and the pressurizer steam space ensures the capability exists to perform this function.

The redundancy design of the Reactor Coolant System vent systems serves to minimize the probability of inadvertent or irreversible actuation while ensuring that a single failure of a vent valve, power supply, or control system does not prevent isolation of the vent path.

The function, capabilities, and testing requirements of the Reactor Coolant System vent system are consistent with the requirements of Item II.b.l of NUREG-0737, "Clarification of TMI Action Plan Requirements," November 1980.