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 FACIL: 50-389 St. Lucie Plant, Unit 2, Florida Power & Light Co.  
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 RECIP. NAME: EISENHUT, D.G. RECIPIENT AFFILIATION: Division of Licensing

DOCKET # 05000389

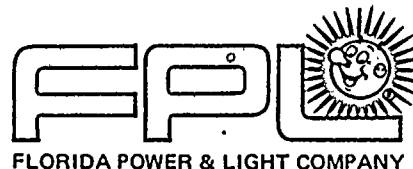
SUBJECT: Forwards list of resolved license conditions for initial criticality. Post-fuel load initial test program initiated, including criticality & low-power testing described in Chapter 14 to FSAR.

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	IE/DEQA/QAB	21	1	1	NRR/DE/AEAB		1	0	
	NRR/DE/CEB	11	1	1	NRR/DE/EHEB		1	1	
	NRR/DE/EQB	13	2	2	NRR/DE/GB	28	2	2	
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	NRR/DL/SSPB		1	0	NRR/DSI/AEB	26	1	1	
	NRR/DSI/ASB		1	1	NRR/DSI/CPB	10	1	1	
	NRR/DSI/CSB	09	1	1	NRR/DSI/ICSB	16	1	1	
	NRR/DSI/METB	12	1	1	NRR/DSI/PSB	19	1	1	
	NRR/DSI/RAB	22	1	1	NRR/DSI/RSB	23	1	1	
	<u>REG FILE</u>	04	1	1	RGN2		3	3	
	RM/DDAMI/MIB		1	0					
EXTERNAL:	ACRS	41	6	6	BNL (AMDTS ONLY)		1	1	
	DMB/DSS (AMDTS)		1	1	FEMA-REP DIV	39	1	1	
	LPDR	03	1	1	NRC PDR	02	1	1	
	NSIC	05	1	1	NTIS		1	1	





May 26, 1983  
L-83-332

Office of Nuclear Reactor Regulations  
Attention: Mr. Darrell G. Eisenhut, Director  
Division of Licensing  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. Eisenhut:

Re: ST. LUCIE UNIT NO. 2  
DOCKET NO. 50-389  
COMPLETION OF LICENSE CONDITIONS  
FOR INITIAL CRITICALITY

NPF-16, St. Lucie Plant Unit No. 2, Facility Operating License contains numerous items that require resolution prior to initial criticality. These items include several license conditions, a license requirement for physical security and an attached list of items to be completed prior to initial criticality. This letter provides the documented resolution of each of these initial criticality items. Accordingly, FPL now considers all licensing actions required for initial criticality resolved and will proceed with the post-fuel load initial test program, including initial criticality and low power testing as described in Chapter 14 of the St. Lucie Unit No. 2 FSAR. If you have any questions or comments regarding this letter, please contact us accordingly.

Very truly yours,

Robert E. Uhrig  
Vice President  
Advanced Systems and Technology

Attachment

REU/RJS/PPC/rms

cc: J.P. O'Reilly, Region II  
Harold F. Reis, Esquire

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Attachment 1

LICENSE CONDITIONS

FOR

INITIAL CRITICALITY

ITEMS IN SECTION 2 OF ST. LUCIE UNIT NO. 2 LICENSE

2.C.16 Initial Test Program (Section 14, SER)

The licensees shall conduct the post-fuel loading initial test program (set forth in Section 14 of the St. Lucie Unit No. 2 Final Safety Analysis Report, as amended through Amendment 13 and FPL's letter, L-83-207) without making any modifications to this program unless such modifications are in accordance with the provisions of 10 CFR Section 50.50. In addition, the licensees shall not make any major modifications to this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- a. Elimination of any test identified as essential in Section 14 of the Final Safety Analysis Report, as amended through Amendment 13 and FPL's letter L-83-207;
- b. Modification of test objectives, methods, or acceptance criteria for any test identified as essential in Section 14 of the Final Safety Analysis Report, as amended through Amendment 13 and FPL's letter L-83-207;
- c. Performance of any test at a power level different from that described in the program as limited by this license authorization, and;
- d. Failure to complete any tests included in the described program (planned or scheduled) for power levels up to the authorized power level.

Status All pre-operational tests required for initial criticality and listed in Chapter 14 and FPL letter L-83-183, dated March 31, 1983, are presently on schedule.

Of the 23 tests FPL listed in FPL letter L-83-174, dated March 28, 1983; 16 are complete. Two (2) are in progress and the remaining tests will be completed by initial criticality.

2.C.17 NUREG-0737 Conditions (Section 22, SER, Section 13.3, SSER 3)

- a. Emergency Response Capability (I.D.1, I.D.2, 13.3.2.8, SSER 3)
  - 1) By April 15, 1983, the licensees shall submit a response to NRC generic letter 82-33, dated December 17, 1982, related to emergency response capabilities.

Status A response to generic letter 82-33, dated December 17, 1982, was sent to the NRC on April 15, 1983 (FPL letter L-83-238). This fulfills the required action for this license condition.

2.C.17 d. Post Accident Sampling System (II.B.3, SSER 3)

Prior to exceeding initial criticality, FPL shall have installed and operational the Post Accident Sampling System.

Status The post-accident sampling system is fully operational.

2.C:17 f. Inadequate Core Cooling Instrumentation (II.F.2, SSER 1)

The licensees shall have:

- 1) The final design core exit thermocouple (CET) system installed with displays in the Qualified Safety Parameter Display System (QSPDS) cabinets, by initial criticality.
- 2) The instrumentation necessary to monitor the display subcooling margin installed in the QSPDS cabinets and operable by initial criticality.

Status 1) The final design core exit thermocouple (CET) system with displays in the Qualified Safety Parameter Display System (QSPDS) cabinets has been installed.

- 2) The instrumentation necessary to monitor and display subcooling margin, in the QSPDS cabinets, has been installed and will be operable by initial criticality.

2.D. The licensees shall fully implement and maintain in effect all the provisions of the Commission-approved physical security, guard training and qualification and safeguards contingency plans including amendments made pursuant to the authority of 10 CFR Section 50.54(p). The approved plans, which contain information described in 10 CFR Section 73.21, are collectively entitled "Florida Power and Light Company, St. Lucie Plant Security Plan," Revision 6, dated August 9, 1982; "St. Lucie Plant Safeguards Contingency Plan" dated June 19, 1980; and "St. Lucie Plant Training and Qualification Plan" dated April 29, 1981, as revised by pages dated June 23, 1981. Prior to initial criticality, the licensees shall satisfy the commitments in FPL letters L-83-215 dated April 5, 1983 and L-83-217 dated April 6, 1983. Also, prior to initial criticality, the licensees shall perform a re-evaluation of vital areas with due regard to the safety-safeguards interface, and provide the NRC the results of their re-evaluation.

Status In FPL letter L-83-293 dated May 11, 1983, implemented the commitments made in FPL letter L-83-215, L-83-217 and during the NRC site visit held on the week of April 18-22, 1983. In addition, L-83-293 re-evaluated the vital areas with due regard to the safety-safeguards interface and also provided the results to the NRC. This completes all FPL action on this license condition.

Items in Attachment 1 of the St. Lucie Unit No. 2 License

I. The licensee will complete to the satisfaction of NRC, Region II, the following open items:

- A. Completion of Fe-55, Sr-89/90 radiochemical analysis procedures (82-72-01)
- B. Modification of particulate sampling collector on plant vent so isotopic analysis can be performed (82-72-03)

- C. Resolve remote shutdown demonstration test followup items (82-63-04)
- D. Resolve undervoltage relay settings for emergency bus transfer (83-30-01)

Status Item I.A - The Sr 89/90 radiochemical analysis has been performed and the results submitted via FPL letter L-83-303 dated May 16, 1983. This completes all required action by FPL.

Item I.B - The necessary modifications to the procedures have been made so that an isotopic analysis can be performed on the sampling collector on the plant vent. (50-389/82-92-03)

Item I.C - FPL provided the NRC with technical discussions concerning the design of the Remote Shutdown Panel via FPL letter L-83-334 dated May 25, 1983. This completes all actions by FPL on this license condition.

Item I.D - The undervoltage relay for emergency bus transfer have been set and the item has been closed to the satisfaction of Region II.

II. The licensee will resolve to the satisfaction of NRC Region II, the following Construction Deficiency Reports (50.55(e) items):

- A. PORV Solenoids Long Term Reliability Deficiency (CDR-83-01).
- B. Westinghouse Gate Valves - Premature Indication of Closure (CDR-83-03).
- C. Boric Acid Makeup (BAM) System Safety Injection Actuation System (SIAS) Pump Start Logic (CDR-83-06).
- D. HPSI Pump Bearing Bracket Drain Deficiency (CDR-83-07).
- E. Condensate Storage Tank Implosion (CDR-83-09).

Status Item II.A - PORV Solenoids Long Term Reliability Deficiency (CDR-83-01).

A final report was submitted to the NRC on March 11, 1983 via FPL letter L-83-144. This item has been closed to the satisfaction of Region II via IE Report 83-37. In addition, all work committed to, in the above letter, has been completed.

Item II.B - Westinghouse gate valve - Premature Indication of Closure (CDR-83-03)

A final report was submitted to the NRC on May 11, 1983 via FPL letter L-83-282. This completes all reporting requirements for FPL.

Item II.C - Boric Acid Makeup (BAM) System Safety Injection Actuation System (SIAS) Pump Start Logic (CDR-83-06).

A final report was submitted to the NRC on April 18, 1983 via FPL letter L-83-243. This item has been closed to the satisfaction of Region II via IE Report 83-37.

Item II.D - HPSI Pump Bearing Bracket Drain Deficiency (CDR-83-07).

A final report was submitted to the NRC on April 18, 1983 via FPL letter L-83-241. This item has been closed to the satisfaction of Region II via IE Report 83-37.

Item II.E - Condensate Storage Tank Implosion (CDR-83-09).

A final report was submitted to the NRC on April 21, 1983 via FPL letter L-83-251. This item has been closed to the satisfaction of Region II via IE Report 83-37.

III. The licensee will inspect and clean as necessary, the electrical cabinets associated with safety-related equipment to verify that conditions are acceptable and consistent with the jumper/lifted lead records.

Status A program has been instituted to ensure that all electrical cabinets associated with safety-related equipment are inspected and cleaned and in addition will verify that conditions are found acceptable and consistent with the jumper/lifted lead records. This is in progress and will be completed prior to initial criticality.



