

## **NRR-PMDAPem Resource**

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**From:** Lingam, Siva  
**Sent:** Wednesday, August 02, 2017 9:21 AM  
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**Cc:** Pascarelli, Robert; Wittick, Brian; Pettis, Robert; dneve@entergy.com; ejohn13@entergy.com; Karipineni, Nageswara  
**Subject:** Grand Gulf - Official RAIs for LAR Associated with One Cycle Extension of Appendix J Type A ILRT and Drywell Bypass Test Interval (CAC No. MF9461)

By letter dated December 29, 2016 (Agencywide Documents Access and Management System Accession No. ML16364A338), Entergy (the licensee) submitted a license amendment request (LAR) to revise the Technical Specifications (TSs) for Grand Gulf Nuclear Station, Unit 1 (GGNS). The LAR would allow for a one cycle extension to the 10-year frequency of the GGNS containment integrated leakage rate test (ILRT) or Type A test and the drywell bypass leak rate test (DWBT). These tests are required by TS 5.5.12, "Containment Leakage Rate Testing Program," and TS Surveillance Requirement 3.6.5.1.1, respectively. The proposed change would permit the existing ILRT and DWBT frequency to be extended from 10 years to 11.5 years.

The U.S. Nuclear Regulatory Commission (NRC) staff from Structural Engineering Branch (ESEB) has reviewed the LAR, and has determined that additional information is required to complete the review. We transmitted the draft request for additional information (RAI) to you by e-mail on July 25, 2017. Subsequently, at your request, we held a clarification conference call on August 1, 2017, discussing the draft RAI and some editorial discrepancies in the LAR. Please note the following **official** RAI, and provide your response within 30 days from the date of this e-mail as mutually agreed. You may also choose to correct the editorial discrepancies in this response. Your timely response will allow the NRC staff to complete its review on schedule.

### **ESEB-RAI-1**

Section 3.3.9 of the LAR provided the results of the most recent (Refueling Outage (RFO)-20) Containment Visual Inspection pursuant to Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, Subsection IWE, but did not provide the results of any of the IWL inspections performed to date. In order for the NRC staff to assess the effective implementation of the licensee's Subsections IWE/IWL Containment Inservice Inspection Program, please provide a discussion of the results for the IWE inspection performed prior to RFO-20, and a discussion of the results for the last two IWL inspection periods.

Siva P. Lingam  
U.S. Nuclear Regulatory Commission  
Project Manager (NRR/DORL/LPL4)  
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