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 AUTH. NAME: UHRIG, R.E. AUTHOR AFFILIATION: Florida Power & Light Co.
 RECIP. NAME: EISENHUT, D.G. RECIPIENT AFFILIATION: Division of Licensing

SUBJECT: Forwards revised test abstract clarifying sections of proposed natural circulation testing plan. Abstract focuses on reducing impact of test & meeting requirements of NUREG-0737, Item I.G.1.

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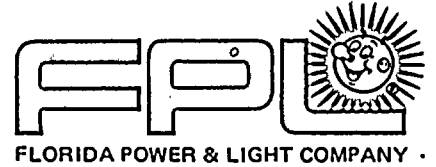
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February 23, 1983
L-83-90

Office of Nuclear Reactor Regulations
Attention: Mr. Darrell G. Eisenhut, Director
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 2055

Dear Mr. Eisenhut:

Re: St. Lucie Unit No. 2
Docket No. 50-389
Natural Circulation Testing Plan
Amendment

In a letter dated January 24, 1983 (Uhrig to Eisenhut L-83-34), Florida Power & Light submitted a test abstract for performing Natural Circulation Testing at the St. Lucie No. 2 during our upcoming Hot Ops II.

The test abstract focused on reducing the impact of this test and also meet the requirements of NUREG-0737-TMI Action Plan I.G.1. Your Mr. W. Long of the Procedures and Test Review Branch has approved the suggested approach. However, in subsequent conversations on February 14, 1983, he requested that we clarify the one time test in which natural circulation will be initiated from a "no flow" condition.

Please find enclosed a revised test abstract which clarifies this section of the proposed test. The enclosed information will be incorporated in a future FSAR amendment.

If you have any further questions with the attached, please contact us.

Very truly yours,

Robert E. Uhrig
Robert E. Uhrig
Vice President
Advanced Systems and Technology

REU/RJS/JES/mp

Attachment

cc: J.P. O'Reilly, Region II
Harold F. Reis, Esquire

Boo!

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PDR ADOCK 05000389
A PDR

NATURAL CIRCULATION TESTING

INITIAL CONDITIONS

- 1) The Reactor Coolant System is at 532° F and 2250 psia, all rods inserted.
- 2) A boron concentration consistent with a 5% shutdown margin at 400°F is present in the Reactor Coolant System.
- 3) Atmospheric steam dump and/or turbine bypass systems are operable.
- 4) Auxiliary feedwater is maintaining steam generator levels within normal range.
- 5) Pressurizer level and pressure controls are in automatic.
- 6) At least one reactor coolant pump in each loop is operating.

TEST SEQUENCE

- 1) Trip all running reactor coolant pumps.*
- 2) Begin cooldown at 25-30°F per hour via atmospheric steam dumps or steam bypass to 400°F.
- 3) Begin reactor coolant system depressurization via auxiliary spray to 1750 psia.
- 4) Stabilize RCS at 400°F and 1750 psia.

TEST RECOVERY

- 1) Start Reactor Coolant Pump(s) in accordance with normal plant procedures.
- 2) Heat up Reactor Coolant system to normal zero power temperature.
- 3) Operate pressurizer heaters to increase reactor coolant pressure to 2250 psia.
- 4) Repeat test sequence and recovery as needed.

*FP&L will perform a one time natural circulation test in which the Reactor Coolant Pumps will be allowed sufficient time to coast down prior to initiating cooldown. Once enough time has elapsed to provide a "no flow" condition, commence cooldown via a single atmospheric dump valve and observe the effect on the "isolated" steam generator.

