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 FACIL: 50-335 St. Lucie Plant, Unit 1, Florida Power & Light Co. 05000335
 AUTH. NAME AUTHOR AFFILIATION
 UHRIG, R. E. Florida Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION
 CLARK, R. A. Operating Reactors Branch 3

SUBJECT: Responds to 820513 request for addl info re responses to IE Bulletin 80-04, "Analysis of PWR Main Steam Line Break W/ Continued Feedwater Addition." No further calculations re magnitude of peak pressure necessary.

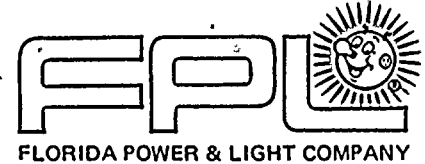
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I have the honor to acknowledge the receipt of your letter of the 14th inst. in relation to the above mentioned matter. I am sorry that I cannot give you a more definite answer at this time, but I am sure that you will understand the necessity of waiting until the proper authorities have had an opportunity to review the matter. I will be glad to advise you again as soon as a final decision has been reached.



August 9, 1982
L-82-335

Office of Nuclear Reactor Regulation
Attention: Mr. Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Clark:

Re: St. Lucie Unit 1
Docket No. 50-335
Main Steam Line Break (MSLB)
With Continued Feedwater Addition

Florida Power & Light has reviewed your letter dated May 13, 1982 concerning our response to IE Bulletin 80-04, "Analysis of a PWR Main Steam Line Break with Continued Feedwater Addition," for St. Lucie Unit 1. Your letter requested additional information concerning the effect of proposed Auxiliary Feedwater (AFW) modifications on containment pressure and on core reactivity. These modifications will allow the AFW system to feed only the steam generator unaffected by a Main Steam Line Break (MSLB).

We have evaluated the effect that the "smart" AFW system will have on containment pressure. Because the system will not open the valves to the faulted steam generator, the automatically initiated AFW flow will not reach it. There will be no AFW contribution to the containment pressure calculation. This will increase the margin between the design pressure and the maximum pressure in the containment during the postulated event. We do not see the need to submit further calculations to provide the magnitude of the peak pressure, because it will be below that which we have previously described in our response to the Bulletin.

Item 2 of your letter concerned core reactivity response to a MSLB. We have contracted with our new fuel vendor to perform a MSLB analysis using the parameters of the "smart" AFW system as input. This analysis is due to be completed in December 1982, at which time we will provide a response to your concern.

This schedule is justified by the fact that the system is not scheduled to be completely installed until our next refueling outage which is presently scheduled for March 1983.

Very truly yours,

R. E. Uhrig
Robert E. Uhrig
Vice President
Advanced Systems & Technology

REU/PLP/mbd

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PDR ADOCK 05000335
Q PDR

cc: Mr. J.P. O'Reilly, Region II
Harold F. Reis, Esquire

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