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AUTH.NAME AUTHOR AFFILIATION
UHRIG,R.E. Florida Power & Light Co.
RECIP.NAME RECIPIENT AFFILIATION
EISENHUT,D.G. Division of Licensing

SUBJECT: Forwards Amend 11 to FSAR.
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P. O. BOX 14000, JUNO BEACH, FL 33408

FLORIDA POWER & LIGHT COMPANY

August 2, 1982
L-82-308

Office of Nuclear Reactor Regulation
Attention: Mr. Darrell G. Eisenhut, Director
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Eisenhut:

Re: St. Lucie Unit 2
Docket No. 50-389
Amendment 11 to The Final
Safety Analysis Report

In accordance with the requirements of 10 CFR 50.30, Florida Power & Light Company (FPL) is submitting sixty (60) copies of Amendment 11 to the Final Safety Analysis Report, St. Lucie Plant Unit 2.

The changes to the amendment are contained in the attachment.

Very truly yours,

Robert E. Uhrig
Vice President
Advanced Systems and Technology

REU/RAK/cab

Attachments

cc: J. P. O'Reilly, Director, Region II (w/o attachments)
Harold F. Reis, Esquire (w/o attachments)

STATE OF FLORIDA)
) ss.
COUNTY OF DADE)

Robert E. Uhrig, being first duly sworn, deposes and says:

That he is Vice President of Florida Power & Light Company, the herein;

That he has executed the foregoing document; that the statements made in this said document are true and correct to the best of his knowledge, information, and belief, and that he is authorized to execute the document on behalf of said

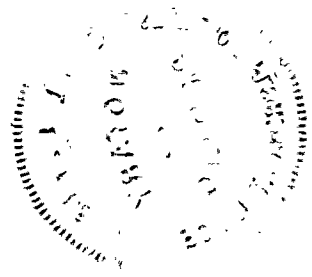
Robert E Uhrig
Robert E. Uhrig

Subscribed and sworn to before me this

2 day of August, 1982

Cheryl I. Fredrick
NOTARY PUBLIC, in and for the County of Dade, State of Florida

My commission expires: Notary Public, State of Florida at Large My Commission Expires October 30, 1983 Bonded thru Maynard Bonding Agency



ATTACHMENT TO

L-82-308 dated August 2, 1982

Changes in the St Lucie Plant Unit 2 FSAR as a result of Amendment 11.

Chapter 1 - The changes basically contain revisions to reflect the changes in plant design as a result of NRC review and various commitments made to the NRC. This makes Chapter 1 consistent with the rest of the FSAR.

In Section 1.1 value of net electrical output for rated power is corrected to correspond to the License Application and ER.

In Section 1.3 slight changes are made to plant parameters to reflect final design values.

In Section 1.7 update of the instrument arrangement and CWD list is provided. In Section 1.9A updates of information related to CE owners group effort on TMI related items are provided.

Chapter 2 - Clarified information found in text, revised references to figures, corrected typographical errors in equations, added references and revised a drawing.

Chapter 3 - Changes are made for consistency with other chapters of the FSAR, provided additional information on jet impingement analysis and recently completed IEALA study (Section 3.9).

In Section 3.10, seismic qualification data for electrical equipment and instrumentation are updated to reflect additional information available from vendor. Section 3.11 writeup for environment qualification has been basically deleted and supplemented with references to the Environmental Qualification Report & Guidebook previously submitted to the NRC under seperate cover.

Chapter 4 - Revised to reflect the fact that part length CEAs will not be used in St Lucie Unit 2. Also updated are core parameters to reflect the final design.

Chapter 5 - Minor clarification, cross-references to other FSAR sections and changes for consistency with other FSAR sections are made.

Chapter 5 (cont'd) - In Section 5.2 missing design information on Reactor Coolant Leak Detection Sensitivity is provided.

In Appendix 5.2B additional information as to the amount of condensate water required to bring St Lucie Unit 2 thru four hours of hotstandby followed by natural circulation cooldown without upper head voiding is provided.

In Section 5.4 Condensate Storage tank water volume is revised to be consistent with already existing Chapter 10 values. The Failure Mode and Effects Analysis for Shutdown Cooling System is reformatted to include additional information on "Inherent Compensating Provisions" and "Remarks and other Efforts".

Section 6.1 - Indicated that insignificant amounts of PVC is used inside the containment and other minor changes to reflect the plant design.

Section 6.2 - Modified design parameters slightly to reflect as-built conditions. Revised location of Hydrogen analyzers, tables in 6.2 to correct typos and other changes for consistency with the information provided to the NRC before. Inserted new revision of figures.

Section 6.3 - Added new information on small break analysis performed with -0.15 and -0.25 ASIU. Safety Injection System FMEA was reformatted. Included new power operated valves to listing.

Section 6.5 - Increased number of TSP baskets present in the vicinity of the SL2 sump.

Section 7.1 - Minor changes for clarification and consistency with information already provided to the NRC.

Section 7.2 - The asymmetric Steam Generator Transient Protective Trip Function is incorporated in the writeup. Reactor Protective System input values are updated to reflect final design.

Section 7.3 - Battery fail bypass is added to the description of AFAS bypasses. Other changes made for clarification and consistency.

Section 7.4 - Post Accident Monitoring Instrumentation listing is updated and "instrument loop accuracys" values are provided. Additions are made to the safety related annunciator windows listing.

Appendix 7.5A - The Safety Assessment System writeup is updated to reflect current design.

Section 7.6 - Added information on Pressurizer Level Alarm and Heater Cutout.

Section 8.2 - Revised subsection on Grid availability to reflect current information, corrected titles on the load flow diagrams.

Section 8.3 - Various changes made to writeup to primarily incorporate changes resulted from NRC review items like QSPDS and SAS. Inserted new Diesel Generator Loading Table and Battery Load Group table showing new loading sequence as a result of NRC concerns and questions.

Chapter 9 - Changes are made to the design information to reflect as built design, for consistency with other FSAR Sections, to incorporate information provided to the NRC previously and for completeness.

Chapter 10 - Inserted latest revision of Drawings where applicable.

Section 10.4 - Provided clarification on the volume of Condensate Storage Tank water available at St Lucie 2. Also, included information on CST water required to hold the plant at hot standby for 23 hours and subsequent cooldown for four hours.

Chapter 11 - Section 11.2 has minor changes made for consistency, clarification and proper cross-references to the other sections of the FSAR and to incorporate information already provided to the NRC.

Designated the Liquid Water Concentrator Bottoms Tank as a future installation.

Section 11.3 - Designated the second oxygen analyzers of the GWMS as a future installation.

Section 11.5 - Updated information on rad monitors per revised vendor's specifications.

Chapter 12 - Upadated radiation monitor data to agree with revised vendor's specifications.

Chapter 13 - Updated to reflect the current personnel changes and restructuring of the organization as a result of NRC review. Plant training schedule is also slightly modified.

Chapter 14 - Discription of Auxiliary Feedwater System functional and endurance test and auto actuation system test are added.

Chapter 15. - This chapter has been revised for consistency with changes made to the other chapters of the FSAR primarily generated as a result of the NRC review.

The sequence of event diagrams are revised to correspond to the new Diesel Generator loading table in Chapter 8.

Revised tables to indicate proper sequencing of events in terms of time and accident assumptions.

Section 15.0 - Added new information on the Atmospheric Dump Valve, Automatic Controls and a statement on the effects of PLCEA on accident analysis. The decision not to use PLCEA's in St Lucie 2 design, does not affect the accident analysis results.

Section 15.C.6 Added new subsection on Inadvertent opening of a pressurizer relief valve.

Section 15.C.7 Added a new subsection on one pump resistance to forced flow.

Section 15.C.8 Added new subsection on opening one Atmospheric Dump Valve.

NRC Questions

210A Revised response to indicate that the list of snubbers would be provided in the Technical Specifications.

NRC Questions

220 Clarified responses to indicate that the pressure described was that of the soil.

NRC Questions

240 Added a clarifying statement to the response to describe the PMH groundwater level used in plant design.

NRC Questions

280 Revised responses to reflect the most recent commitments made to the NRC.

NRC Questions

410 — Inserted new response on Light Loads. Information has been previously transmitted to the NRC. Revised response due to the results of flooding of safety related equipment.

NRC Questions

420 — Included additional information developed as a result of various NRC review requirements. Regulatory Guide 1.97 listing is updated to add new items.

NRC Questions

430A — Inserted Question/Responses on PSB, R.G. 1.63 conformance and total loss of A.C. Power. This information has been previously transmitted to the NRC.

NRC Questions

440 — Clarifications added and changes made to incorporate requirements resulting from NRC review.