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ACCESSION NBR: 8104290476 DOC. DATE: 81/04/27 NOTARIZED: YES  
 FACIL: 50-389 St. Lucie Plant, Unit 2, Florida Power & Light Co.  
 AUTH. NAME: UHRIG, R.E. AUTHOR AFFILIATION: Florida Power & Light Co.  
 RECIPIENT AFFILIATION: Division of Licensing  
 EISENHUT, D.G.

DOCKET # 05000389

SUBJECT: Forwards Amend 2 to FSAR, providing revised Chapter 15 re accident analysis & App 15A.

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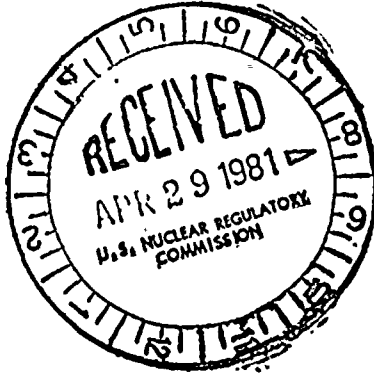
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	LIC BR #3 LA	1	0	NERSES, V. 04	1	1
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	CHEM ENG BR 08	1	1	CONT SYS BR 09	1	1
	CORE PERF BR 10	1	1	EFF TR SYS BR12	1	1
	EMERG PREP 22	1	0	EQUIP QUAL BR13	3	3
	GEOSCIENCES 14	1	1	HUM FACT ENG BR	1	1
	HYD/GEO BR 15	2	2	I&C SYS BR 16	1	1
	I&E. 06	3	3	LIC GUID BR	1	1
	LIC QUAL BR	1	1	MATL ENG BR 17	1	1
	MECH ENG BR 18	1	1	MPA	1	0
	NRC PDR 02	1	1	OELD	1	0
	OP LIC BR	1	1	POWER SYS BR 19	1	1
	PROC/TST REV 20	1	1	QA BR 21	1	1
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EXTERNAL:	ACRS 27	16	16	LPDR 03	1	1
	NSIC 05	1	1			

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APPY

JF





April 27, 1981  
L-81-180

Office of Nuclear Reactor Regulation  
Attention: Mr. Darrell G. Eisenhut, Director  
Division of Licensing  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

Dear Mr. Eisenhut:

Re: St. Lucie Unit No. 2  
Docket No. 50-389  
Amendment 2 to the Final Safety Analysis Report

In accordance with the requirements of 10 CFR 50.30, Florida Power & Light Company is submitting sixty (60) copies of Amendment 2 to the Final Safety Analysis Report, St. Lucie Plant Unit No. 2.

This amendment provides a revised Chapter 15, Accident Analysis, and adds Appendix 15A. The amendment includes the following:

1. Editorial changes have been made to the text and the Sequence of Events Diagrams to improve consistency and ease of review.
2. The reference to Appendix 15C for the time-at-temperature fuel failure criterion used in seized pump shaft analysis has been removed from Section 15.0.4.4.3. In the seized pump shaft analysis writeups, the predicted maximum cladding temperature has been increased from 1310°F to 1500°F and the percentage of fuel experiencing DNB has been increased from 3% to 5% for the LF-1 event (15.3.3.3) and from 9.4% to 13% for the LF-2 event (15.3.4.3).
3. The analysis of an open letdown relief valve in Section 15.6.5.1 has been replaced with the analysis of a letdown line break. The radiological doses for the letdown line break are less severe than those for the open relief valve and the conclusion with respect to radiological effects remains unchanged.
4. Section 15.8 has been revised to include analysis of a failure in the low temperature overpressure protection system.

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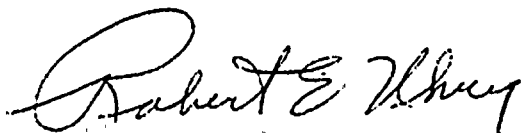
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Mr. Darrell G. Eisenhut, Director  
Division of Licensing  
U. S. Nuclear Regulatory Commission

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5. Large steam line break analyses which correspond to the Standard Review Plan, Section 15.1.5, are provided in Appendix 15A.

Very truly yours,

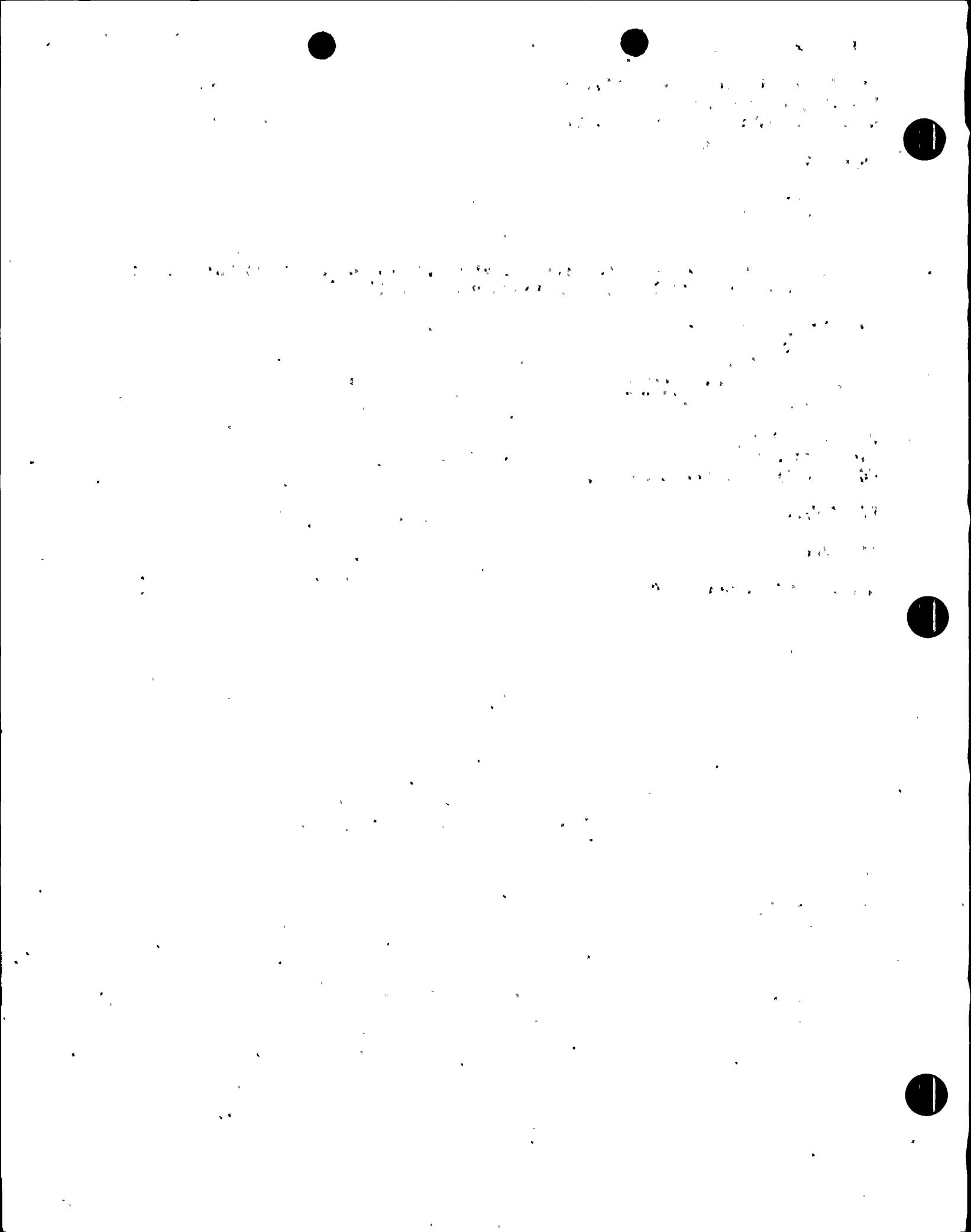


Robert E. Uhrig  
Vice President  
Advanced Systems and Technology

REU/TCG/gw

Attachments

cc: Harold F. Reis, Esquire



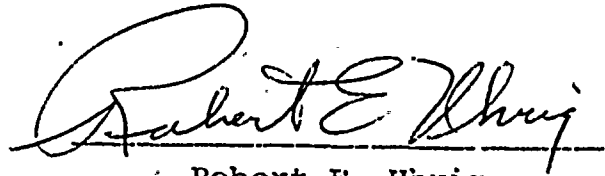
STATE OF FLORIDA )  
                          )  
COUNTY OF DADE )

ss.

Robert E. Uhrig, being first duly sworn, deposes and says:

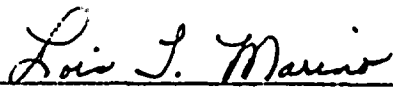
That he is a Vice President of Florida Power & Light Company,  
the Licensee herein;

That he has executed the foregoing document; that the state-  
ments made in this said document are true and correct to the  
best of his knowledge, information, and belief, and that he  
is authorized to execute the document on behalf of said  
Licensee.

  
Robert E. Uhrig

Subscribed and sworn to before me this

27 day of April, 1981

  
\_\_\_\_\_  
NOTARY PUBLIC, in and for the county of Dade,  
State of Florida

My commission expires: \_\_\_\_\_

NOTARY PUBLIC STATE OF FLORIDA at LARGE  
MY COMMISSION EXPIRES AUGUST 24, 1981  
BONDED THRU MAYNARD BONDING AGENCY

