



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

MAY 8 1980

In Reply Refer To:

RII:JPO

50-335, 50-389

50-250, 50-251

Florida Power and Light Company
Attn: R. E. Uhrig, Vice President
Advanced Systems and Technology
P. O. Box 529100
Miami, Florida 33152

Gentlemen:

The enclosed Information Notice provides early notification of an event that may have safety significance. Accordingly, you should review the Information Notice for possible applicability to your facility.

No specific action or response is requested at this time; however, contingent upon the results of further staff evaluation, a Bulletin or Circular recommending or requesting specific licensee actions may be issued. If you have questions regarding this matter, please contact me.

Sincerely,



James P. O'Reilly
Director

Enclosures:

1. IE Information Notice
No. 80-20
2. List of Recently Issued
IE Information Notices

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Florida Power and
Light Company

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cc w/encl:

C. M. Wethy, Plant Manager
Post Office Box 128
Ft. Pierce, Florida 33450

Nat Weems, Assistant QA Manager
Post Office Box 128
Ft. Pierce, Florida 33450

H. E. Yaeger, Plant Manager
Post Office Box 013100
Miami, Florida 33101



UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D. C. 20555

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May 8, 1980

IE Information Notice 80-20

LOSS OF DECAY HEAT REMOVAL CAPABILITY AT DAVIS-BESSE UNIT 1 WHILE IN A
REFUELING MODE

Description of Circumstances:

On April 19, 1980, decay heat removal capability was lost at Davis-Besse Unit 1 for approximately two and one-half hours. At the time of the event, the unit was in a refueling mode (e.g., RCS temperature was 90F; decay heat was being removed by Decay Heat Loop No. 2; the vessel head was detensioned with bolts in place; the reactor coolant level was slightly below the vessel head flanges; and the manway covers on top of the once through steam generators were removed). (See Enclosure A, Status of Davis-Besse 1 Prior to Loss of Power to Busses E-2 and F-2 for additional details regarding this event.)

Since the plant was in a refueling mode, many systems or components were out of service for maintenance or testing purposes. In addition, other systems and components were deactivated to preclude their inadvertent actuation while in a refueling mode. Systems and components that were not in service or deactivated included:

- Containment Spray System;
- High Pressure Injection System;
- Source Range Channel 2;
- Decay Heat Loop No. 1;
- Station Battery 1P and 1N;
- Emergency Diesel-Generator No. 1;
- 4.16 KV Essential Switchgear Bus C1; and
- 13.8 KV Switchgear Bus A (this bus was energized but not aligned).

In brief, the event was due to the tripping of a non-safeguards feeder breaker in 13.8 KV Switchgear Bus B. Because of the extensive maintenance and testing activities being conducted at the time, Channels 1 and 3 of the Reactor Protection System (RPS) and Safety Features Actuation System (SFAS) were being energized from only one source, the source emanating from the tripped breaker. Since the SFAS logic used at Davis-Besse is a two-out-of-four input scheme in which the loss (or actuation) of any two input signals results in the actuation of all four output channels (i.e., Channels 1 and 3, and Channels 2 and 4), the loss of power to Channels 1 and 3 bistables also resulted in actuation of SFAS Channels 2 and 4. The actuation of SFAS Channels 2 and 4, in turn, affected Decay Heat Loop No. 2, the operating loop.

Since the initiating event was a loss of power event, all five levels of SFAS were actuated (i.e., Level 1 - High Radiation; Level 2 - High Pressure Injection; Level 3 - Low Pressure Injection; Level 4 - Containment Spray; and

Level 5 - ECCS Recirculation Mode). Actuation of SFAS Level 2 and/or 3 resulted in containment isolation and loss of normal decay heat pump suction from RCS hot leg No. 2. Actuation of SFAS Level 3 aligned the Decay Heat Pump No. 2 suction to the Borated Water Storage Tank (BWST) in the low pressure injection mode. Actuation of SFAS Level 5 represents a low level in the BWST; therefore, upon its actuation, ECCS operation was automatically transferred from the Injection Mode to the Recirculation Mode. As a result, Decay Heat Pump No. 2, the operating pump, was automatically aligned to take suction from the containment sump rather than from the BWST or the reactor coolant system. Since the emergency containment sump was dry, suction to the operating decay heat pump was lost. As a result, the decay heat removal capability was lost for approximately two and one-half hours, the time required to vent the system. Furthermore, since Decay Heat Loop No. 1 was down for maintenance, it was not available to reduce the time required to restore decay heat cooling.

MAJOR CONTRIBUTORS TO THE EVENT:

The rather extended loss of decay heat removal capability at Davis-Besse Unit 1 was due to three somewhat independent factors, any one of which, if corrected, could have precluded this event. These three factors are:

- (i) Inadequate procedures and/or administrative controls;
- (ii) Extensive maintenance activities; and
- (iii) The two-out-of-four SFAS logic.

Regarding inadequate procedures and/or administrative controls, it should be noted that the High Pressure Injection Pumps and the Containment Spray Pumps were deactivated to preclude their inadvertent actuation while in the refueling mode. In a similar vein, if the SFAS Level 5 scheme had been by-passed or deactivated while in the refueling mode, or if the emergency sump isolation valves were closed and their breakers opened, this event would have been, at most, a minor interruption of decay heat flow.

Regarding the extensive maintenance activities, it appears that this event would have been precluded, or at least ameliorated, if the maintenance activities were substantially reduced while in the refueling mode. For example, if the maintenance activities had been restricted such that two SFAS channels would not be lost by a single event (e.g., serving Channels 1 and 3 from separate sources), this event would have been precluded. Likewise, if maintenance activities had been planned or restricted such that a backup decay heat removal system would have been readily available, the consequences of the loss of the operating decay heat removal loop would have been ameliorated.

Regarding the two-out-of-four SFAS logic used at Davis-Besse, even under normal conditions, it appears that this type of logic is somewhat more susceptible to spurious actions than other logic schemes (e.g., a one-out-of-two taken-twice scheme). This susceptibility is amplified when two SFAS channels are served from one source. Consequently, when the source feeding SFAS Channels 1 and 3 was lost, all five levels of SFAS were actuated. As stated

previously, this particular event would have been precluded if SFAS Channels 1 and 3 were being served from separate and independent sources. In a similar vein, this specific event would have been precluded by a one-out-of-two taken twice type of logic that requires the coincident actuation of or loss of power of an even numbered SFAS Channel and an odd numbered SFAS Channel.

Since each LWR can be expected to be in a refueling mode many times during its lifetime, licensees should evaluate the susceptibility of their plants to losing decay heat removal capability by the causes described in this Information Notice. No specific action or response is requested at this time. Licensees having questions regarding this matter should contact the director of the appropriate NRC Regional Office.

Enclosure:
Davis-Besse Event of
April 19, 1980

DAVIS-BESSE EVENT OF APRIL 19, 1980

STATUS OF DAVIS-BESSE 1 PRIOR TO LOSS OF POWER TO BUSES E-2 AND F-2:

1. Refueling mode with RCS temperature at 90° F and level slightly below vessel head flange. Head detensioned with bolts in place. Manway cover on top of OTSG removed. Tygon tubing attached to lower vents of RCS hot leg for RCS level indication. Decay heat loop 2 in service for RCS cooling.
2. All non-nuclear instrument (NNI) power and Static Voltage Regulator YAR supplied from 13.8 KV Bus B via HBBF2. 13.8 K Bus A energized but not connected. RPS and SFAS Channels 1 and 3 being supplied from YAR.
3. Equipment Out of Service
 - a. Source Range Channel 2 - Surveillance
 - b. Emergency Diesel Generator 1 - Maintenance.
 - c. Decay Heat Loop 1 - Maintenance.
4. Breakers for containment spray and HPI pumps racked out.

SEQUENCE OF EVENTS

<u>TIME</u>	<u>EVENT</u>	<u>CAUSE/COMMENTS</u>
2:00 p.m.	Loss of power to Busses E-2 and F-2 (non-essential 480 VAC)	Ground short on 13.8 KV breaker HBBF2 which caused breaker to open. This interrupted power to busses E-2 and F-2 which were supplying all non-nuclear instrument (NNI) power, channels 1 and 3 of the Reactor Protection System (RPS) and the Safety Features Actuation Signal (SFAS), the computer, and much of the control room indicators.
2:00 p.m.	SFAS Level 5 (recirculation mode) actuation.	Two out of four logic tripped upon loss of Busses E-2 and F-2. Actuation caused ECCS pump suction valves from containment sump to open and ECCS pump suction valves from Borated Water Storage Tank to close. During valve travel times, gravity flow path existed from BWST to containment sump.
2:02 p.m.	Decay Heat (low pressure safety injection) flow secured by operator	Operator turned off only operating DH pump to avoid spillage of RCS water to containment via the tygon tubing for RCS level indication and open SG manway.
2:33 p.m.	Partial restoration of power	Power to Bus E-2 and SFAS channels 1 and 3 restored along with one channel of NNI. This restored all essential power for ECCS.

<u>TIME</u>	<u>EVENT</u>	<u>CAUSE/COMMENTS</u>
2:44 p.m.	Attempt to reestablish DH flow	Started DH pump 1-2 then stopped it when it was determined that air was in suction line. Pump secured to prevent damage.
3:34 p.m.	Source Range Channel 2 energized.	
4:00 p.m. to 4:06 p.m.	Restoration of Busses (480 VAC) F-2, F-21, F-22, and F-23	Busses restored sequentially as efforts progressed to isolate ground fault.
4:25 p.m.	DH flow restored	DH pump 1-2 started after venting. RCS temperature at 170°F. DH flow bypassing cooler. Incore TC's being taken and maximum is 170°F.
4:46 p.m.	Containment sump pump breakers opened	Precautionary measure to assure containment sump water from BWST remained in containment. Incore TC's range from 161 to 164°F.
5:40 p.m.	Computer returned to service.	Incore TC's range from 158 to 160°F.
6:24 p.m.	DH flow directed through cooler	RCS cooldown established at less than 25°F per hour. RCS temperature at 150°F. Incore TC's range from 151 to 158°F.
9:50 p.m.	Power completely restored	RCS temperature at approximately 115°F.

STATUS OF DAVIS-BESSE 1 AFTER RECOVERY FROM LOSS OF POWER TO BUSES E-2 AND F-2:

1. Refueling mode with RCS temperature at 115°F and level slightly below vessel head flange. Head detensioned with bolts in place. Manway cover on top of OTSG removed. Tygon tubing attached to lower vents of RCS hot leg for RCS level indication. Decay heat loop 2 in service for RCS cooling.
2. Bus E-2 being supplied from 13.8 KV Bus A via breaker HAAE2 and Bus F-2 being supplied from 13.8 KV Bus B via breaker HBBF2.
3. Decay heat loop filled, all tags clear. Maintenance work restricted so restoration of system will be less than two hours.
4. ECCS pump suction valves (DH-9A and DH-9B) from containment sump closed and breakers racked out. This will prevent the suction of air into the decay

heat loop during a level 5 actuation (recirculation mode) when there is no water in the sump.

5. Equipment Out of Service:

Emergency Diesel Generator 1 - maintenance

6. Breakers for containment spray and HPI pumps racked out.

IE Information Notice No. 80-20
May 8, 1980

Enclosure

RECENTLY ISSUED
IE INFORMATION NOTICES

Information Notice No.	Subject	Date Issued	Issued To
80-20	Loss of Decay Heat Removal Capability at Davis-Besse Unit 1 While in a Refueling Mode	5/8/80	All holders of power reactor operation Licenses or construction permit
80-19	NIOSH Recall of Recirculating-Mode (Closed-Circuit) Self-Contained Breathing Apparatus (Rebreathers)	5/6/80	All holders of a power reactor OL, Research Reactor License, Fuel Cycle Facility License and Priority I Material License
80-18	Possible Weapons Smuggling Pouch	5/5/80	All power reactor facilities with an OL, fuel fabrication and processing facilities and Materials Priority I licensees (processors and distributors)
80-17	Potential Hazards Associated With Interchangeable Parts On Radiographic Equipment	5/5/80	All radiography Licenses
80-16	Shaft Seal Packing Causes Binding In Main Steam Swing Check And Isolation Valves	4/29/80	All power reactor facilities in your Region with an OL or CP
80-15	Axial (Longitudinal) Oriented Cracking In Piping	4/21/80	All Light Water Reactor Facilities holding power reactor OLs or CPs
80-14	Safety Suggestions From Employees	4/2/80	All power reactor facilities with an OL or CP
80-13	General Electric Type SBM Control Switches - Defective Cam Followers	4/2/80	All light water reactor facilities holding power reactor OLs or CPs
80-12	Instrument Failure Causes Opening of PORV and Block Valve	3/31/80	All holders of power reactor OLs and CPs
80-11	General Problems with ASCO Valves in Nuclear Application Including Fire Protection Systems	3/14/80	All holders of Reactor OL, CP, fuel fabrication and processing facilities