

ACCESSION NBR: 7909040506 JC. DATE: 79/08/29 NOTARIZ: NO  
FACIL: 50-255 Palisades Nuclear Plant, Consumers Power Co.  
50-285 Fort Calhoun Station, Unit 1, Omaha Public Power Dist  
50-309 Maine Yankee Atomic Power Plant, Maine Yankee Atomic  
AUTH. NAME AUTHOR AFFILIATION  
LIEBLER, G.E. C-E Power Systems  
RECIP. NAME RECIPIENT AFFILIATION  
STELLO, V. Office of Inspection & Enforcement

DOCKET #  
05000255  
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368  
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SUBJECT: Forwards proprietary version of CEN-115-P, response to IE  
Bulletin 79-06C items 2 & 3, Response withheld (ref  
10CFR2.790).

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August 29, 1979

Mr. Victor Stello, Director  
Office of Inspection & Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

PROPRIETARY DOCKET FILE COPY

Subject: Thirty Day Response to IE Bulletin 79-06C

Dockets: 50-335 (FP&L), 50-317 and 50-318 (BG&E), 50-368 (AP&L),  
50-309 (MY), 50-336 (NE Utilities), 50-255 (CPCo.) and  
50-285 (OPPD)

- Enclosures: (1) CEN-115-P, Response to NRC IE Bulletin 79-06C Items 2 and 3  
for Combustion Engineering Nuclear Steam Supply Systems  
(Proprietary version, copy #0000/ 6 through 000020)
- (2) CEN-115-NP (Non Proprietary version of Enclosure (1))
- (3) Proprietary Information Affidavit for CEN-115-P

Dear Mr. Stello:

Enclosures (1) and (2) are forwarded for your review. Please note that this report constitutes the response to the short term action items number (2) and (3) of IE Bulletin 79-06C. The information presented in CEN-115-P is generic in nature and applies to all operating C-E designed reactors.

The results of analysis contained in the attached report indicate the need to trip reactor coolant pumps within approximately 10 minutes for a small subset of small break LOCA's. In response to Long Term Action Item 1 of IE Bulletin 79-06C, we consider it premature to propose a design for automatic tripping of RCPs. The bases for this conclusion are:

- (1) Best Estimate calculations with both HPSI pumps show no need for pump tripping.
- (2) Analytical modeling of these events is evolving and may alter conclusions.
- (3) The identified small breaks of concern represent a small subset of the entire break spectrum.
- (4) ~~Other design alternates exist to provide equivalent core protection.~~

It is judged that the guidelines provided in response to Bulletin Item 3 provide adequate core protection. We will continue to closely monitor all developments on this subject so that these guidelines continue to ensure safe plant operation.

NONE  
1/10  
EXCL # 2  
NOT PROVIDED

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Due to the proprietary nature of some of the material contained in CEN-115-P, we request that it be withheld in accordance with provisions of 10 CFR 2.790 and that this material be safeguarded. The reasons for the classification of this material as proprietary are delineated in the enclosed affidavit.

Five copies of both Enclosures (1) and (2) are included with this letter for immediate delivery. Thirty-five additional copies of Enclosure (1) and fifteen copies of Enclosure (2) will be forwarded via the regular mail service.

Should questions arise concerning this report, please feel free to contact myself at (305) 552-3811 or Mr. Ken Morris of Omaha Public Power District at (402) 536-4504.

Sincerely,

*Robert T. Harris for*  
George E. Liebler, Chairman  
C-E Owners Group

cc: Dr. D. F. Ross, Manager (w/o attach)  
Bulletins & Orders Task Force

✓ Mr. I. Villalva (with attach)

Dr. Harold Denton (w/o attach)

50-255/285/309  
DOCKET NO. 335/317/318/368  
DATE: 9-5-79 336

NOTE TO NRC AND/OR LOCAL PUBLIC DOCUMENT ROOMS

The following item submitted with letter dated 8-29-79  
from CE is being withheld from public  
disclosure in accordance with Section 2.790.

PROPRIETARY INFORMATION

CEN-115-P. Response TO NRC IE  
BULLETIN 79-06C  
   
   
 

Dow Lawham  
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