

NuScaleDCRaisPEm Resource

From: Cranston, Gregory
Sent: Tuesday, July 25, 2017 9:32 AM
To: RAI@nuscalepower.com
Cc: NuScaleDCRaisPEm Resource; Lee, Samuel; Chowdhury, Prosanta; Li, Chang; Markley, Anthony; Dias, Antonio
Subject: RE: Request for Additional Information No. 104, RAI 8918 (9.2.7)
Attachments: Request for Additional Information No. 104 (eRAI No. 8918).pdf

Attached please find NRC staff's request for additional information concerning review of the NuScale Design Certification Application.

Please submit your technically correct and complete response within 60 days of the date of this RAI to the NRC Document Control Desk.

If you have any questions, please contact me.

Thank you.

Gregory Cranston, Senior Project Manager
Licensing Branch 1 (NuScale)
Division of New Reactor Licensing
Office of New Reactors
U.S. Nuclear Regulatory Commission
301-415-0546

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From: Cranston, Gregory

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Request for Additional Information No. 104 (eRAI No. 8918)

Issue Date: 07/25/2017

Application Title: NuScale Standard Design Certification - 52-048

Operating Company: NuScale Power, LLC

Docket No. 52-048

Review Section: 09.02.07 - Site Cooling Water System (NuScale SMR design)

Application Section: 9.2

QUESTIONS

09.02.07-1

10 CFR 52.47(a)(2) requires that a standard design certification applicant provide a description and analysis of the structures, systems, and components (SSCs) of the facility, with emphasis upon performance requirements, the bases, with technical justification therefor, upon which these requirements have been established, and the evaluations required to show that safety functions will be accomplished.

GDC 4 requires, in part, that SSCs important to safety "shall be designed to accommodate the effects of and to be compatible with the environmental conditions associated with normal operation, maintenance, testing, and postulated accidents, including loss-of-coolant accidents. These structures, systems, and components shall be appropriately protected against dynamic effects, including the effects of missiles, pipe whipping, and discharging fluids, that may result from equipment failures and from events and conditions outside the nuclear power unit."

FSAR Tier 2, Section 9.2.7.1, "Design Bases," states that GDC 4 was considered in the design of the site cooling water system (SCWS). No further information is provided to substantiate such statement.

The applicant is requested to describe how GDC 4 was considered in the SCWS.