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JAN 5 1977

Docket Nos. (50-237, 50-249, 50-254, 50-265)

Commonwealth Edison Company  
 ATTN: Mr. R. L. Bolger  
 Assistant Vice President  
 Post Office Box 767  
 Chicago, Illinois 60690

Gentlemen:

Your submittals dated October 28, 1976, requested amendments to Facility Operating Licenses DPR-19, DPR-25, DPR-29 and DPR-30 relative to operation of the Dresden and Quad Cities Nuclear Power Stations with one automatic depressurization valve out of service for up to 30 days.

We are reviewing your submittal and have determined that the additional information requested in Enclosure A is necessary to continue our review.

To enable us to maintain our review schedule, please submit the requested information within 45 days of receipt of this letter.

Sincerely,

Original signed by  
 Dennis L. Ziemann  
 Dennis L. Ziemann, Chief  
 Operating Reactors Branch #2  
 Division of Operating Reactors

Enclosure:  
 Request for Additional  
 Information

cc w/enclosure:  
 See next page

*Handwritten initials and marks:* [initials], [initials], [initials], [initials]

OFFICE >	DOR:ORB #2	DOR:ORB #2	DOR:ORB #2		
SURNAME >	PO'Connor:ro	RDSilver	DLZiemann		
DATE >	1 15 1977	1 15 1977	1 15 1976		

JAN 5 1977

cc w/enclosure:

Mr. Charles Whitmore  
President and Chairman  
Iowa-Illinois Gas and  
Electric Company  
206 East Second Avenue  
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Moline Public Library  
504 - 17th Street  
Moline, Illinois 61265

Morris Public Library  
604 Liberty Street  
Morris, Illinois 60451

ENCLOSURE A

REQUEST FOR ADDITIONAL INFORMATION

DRESDEN STATION UNIT NOS. 2 AND 3

QUAD CITIES STATION UNIT NOS. 2 AND 3

DOCKET NOS. 50-237, 50-249, 50-254, 50-265

1. The ECCS analysis assumed that "the most limiting ADS valve (is) out of service". Describe what is meant by the most limiting valve. Identify the limiting valve for each of the four plants covered by this analysis.
2. Provide additional ECCS analyses, with the limiting ADS valve out of service, of small breaks in the break area range  $.10 \text{ ft}^2$  to  $.03 \text{ ft}^2$  as well as additional analyses in the break area range  $.005 \text{ ft}^2$  to  $.03 \text{ ft}^2$  to assure that the maximum peak clad temperature has been determined.
3. For the breaks already analysed and the breaks to be analysed in connection with Item 2 above, furnish the following information vs. time:
  - (a) Peak Cladding Temperature
  - (b) Reactor Vessel Pressure
  - (c) Water Level inside the core shroud
  - (d) Convective Heat Transfer Coefficient
4. For the breaks already analysed and the breaks to be analysed in connection with Item 2 above, provide the peak clad local oxidation percent and the core-wide metal-water reactor percent.
5. The abnormal operating transient analyses in the reload submittals assume that all relief valves are operable. Provide an assessment of the maximum effect on the  $\Delta\text{MCPR}$  for the most limiting abnormal operational transient with the most limiting relief valve being inoperative.