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TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [XX]

The high contact resistance on normally open contact XK30 XKF188-1 prevented the contact from closing when control relay XK30 XKF188 was energized (test pushbutton SV2 depressed). Consequently, relay K6D19 KF190 remained de-energized, its associated normally closed contact remained closed, and relays XK16 and XK17 remained energized. As a result, when #2 TSV reached 5% closed, the other three TSVs began closing and initiated a reactor scram.

The main generator and thus the main turbine did not automatically trip due to component failure. The Operational Analysis Department (OAD) inspected the main generator reverse power relay per Work Request 84140 and found dirt located between the bearing and contact pivot arm on the relay directional unit. The dirt caused mechanical binding of the contact pivot arm which prevented the contact from operating as required.

D. SAFETY ANALYSIS OF EVENT:

The initiating action for this event was the partial closure of all four turbine stop valves. Any combination of three or more TSVs each closed 10% or greater during normal power operation at greater than 45% will result in a reactor scram. The reactor scram occurs in anticipation of the pressure, neutron flux, and fuel cladding surface heat flux increase caused by the rapid closure of the TSVs and a failure of the turbine bypass valves to open. A reactor scram was the response to the TSV closures of this event. Additionally, the turbine bypass valves responded properly to the TSV closure.

The main generator protective relaying is designed such that two circuits will independently trip the generator on reverse power. The first trip circuit, associated with a primary reverse power relay, is initiated following a turbine trip signal. This circuit is designed to trip the generator at -1.3 MWe after a five second time delay. The second trip circuit is associated with a secondary reverse power relay and it is designed to trip the main generator at -1.3 MWe with a 15 second time delay in the absence of a turbine trip signal. Thus, the secondary reverse power relay should have initiated the generator trip since there were no turbine trip signals present. Although the secondary reverse power relay failed during this event, the primary reverse power relay was available in the event of a turbine trip. Also, the manual capability of tripping the turbine operated successfully thereby preventing damage to the generator.

For the above reasons, the safety significance of this event was considered to be minimal.

E. CORRECTIVE ACTIONS:

Corrective actions for this event involved testing of control relays XK30 XKF188 and K6D19 KDF189. Although the only problem discovered was the contact high resistance on relay XK30 XKF188, both relays were replaced. This work was completed on April 18, 1989 under Work Request 84112.

Corrective actions for the secondary reverse power relay failure involved the removal of dirt found between the bearing surface and the contact pivot arm surface. After removing the dirt between the surfaces, the relay operated successfully. The reverse power relays are inspected and calibrated on a refueling outage basis in accordance with the OAD Protective Relay Calibration Procedure. No problems were discovered during the previous inspection. To prevent future failures of this type, the relay calibration procedure will be revised to clarify the physical inspection section such that mechanical binding of the relay pivot arm is specifically addressed. This revision will be tracked through completion by a commitment made in LER 89-002/050249 (249-200-89-02005).

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Prior to rolling the main turbine during Unit startup in accordance with Dresden General Procedure (DGP) 1-1, Unit 2(3) Normal Unit Startup, all of the turbine control valves, stop valves, and combined intermediate valves were functionally tested in accordance with DOS 5600-2 and DOS 500-10, Turbine Stop Valve Closure Scram Circuit Functional Test. As Unit startup continued, the same turbine valves were again functionally tested once reaching 1800 rpm but prior to synchronizing the main generator to the grid. Once the main generator was synchronized to the grid; all turbine valves were again functionally tested just prior to reaching 40% rated core thermal power. Finally, reactor power output was administratively limited to 40% until all surveillance testing results were completely reviewed to ensure that no problems existed within the control circuitry of the turbine control valves, stop valves, and combined intermediate valves.

F. PREVIOUS OCCURRENCES:

<u>LER/Docket Number</u>	<u>Title</u>
89-002/050249	Reactor Scram Due to Failure of an Electrical Protection Assembly Breaker. A reactor scram resulted due to the failure of an Electrical Protection Assembly (EPA) breaker. However, this event also involved the failure of the main generator to trip on reverse power with approximately -20 MWe indicated on the Center Desk display. The cause of this event became evident during the investigation performed for LER 89-006/050249.
86-025/050249	Unit 3 Turbine Stop Valve 10% Closure Scram Due to Limit Switch Contact Failure. This report documented a reactor scram due to component failure. Investigation of this event determined that limit switch SVOS-2 had momentarily opened and then reclosed as a result of the contacts being carbonized. This resulted in the closure of the TSVs. The corrective actions for this event involved replacement of limit switch SVOS-2.

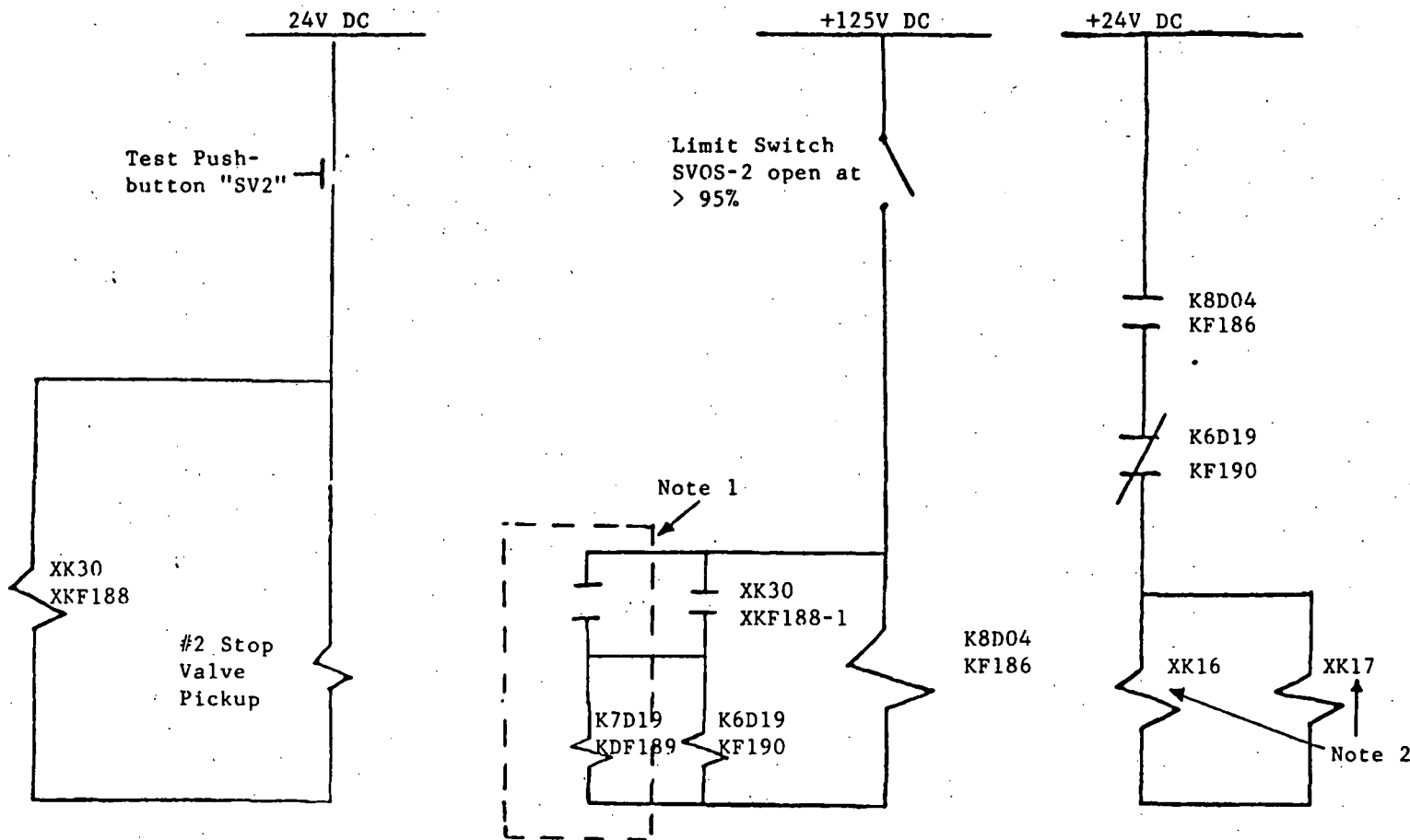
G. COMPONENT FAILURE DATA:

<u>Manufacturer</u>	<u>Nomenclature</u>	<u>Model Number</u>	<u>Mfg. Part Number</u>
General Electric	Control Relay	CR120HF47J10A	N/A
General Electric	Reverse Power Relay	GGP53C	N/A

An industry wide NPRDS component search was performed and there were no reported previous failures of either of these two types of relays.

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Note 1: This contact is associated with seal-in circuit relay K7D19 KF189 and closes when the relay is energized.

Note 2: These relays when energized cause #1, 3 and 4 turbine stop valves to follow the action of #2 turbine stop valve.

TURBINE STOP VALVE TEST LOGIC

Figure 1



Commonwealth Edison

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/942-2920

May 15, 1989

EDE LTR #89-391

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Licensee Event Report #89-006-0, Docket #050249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73(a)(2)(iv).

E.D. Eenigenburg
Station Manager
Dresden Nuclear Power Station

EDE/ade

Enclosure

cc: A. Bert Davis, Regional Administrator, Region III
File/NRC
File/Numerical

(0573k)

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