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June 17, 1988

Mr. Thomas E. Murley, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Dresden Station Unit 3
Cycle 11 Reload Amendment
NRC Docket No. 50-249

Reference: Letter from J.A. Silady to T.E. Murley
dated March 9, 1988.

Dear Mr. Murley:

Enclosed are two pages which were inadvertently omitted from Enclosure E of the referenced reload licensing submittal during the distribution process. The summary and significant hazards evaluations, description of proposed changes, transmittal letter and associated bases page all properly reflected the changes involved in the omitted pages.

The omission was identified during a final review by CECO of the reload amendment page changes at the request of the NRR Project Manager. The additional review prior to issuance was performed since this is the first Dresden amendment which is being issued based on a transfer of Appendix A specifications from CECO's word processing files to NRR's system.

During the review, a number of discrepancies were identified by CECO in the NRR draft version of this amendment. These have also been discussed with the Project Manager on June 16 and 17, 1988.

Please contact this office should further information be required.

Very truly yours,

J. A. Silady
Nuclear Licensing Administrator

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Enclosure

cc: A.B. Davis - Regional Administrator, RIII
B.L. Siegel - Project Manager, NRR
S.G. DuPont - Senior Resident Inspector, Dresden
M.C. Parker - IDNS

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3.5 LIMITING CONDITION FOR OPERATION
(Cont'd.)

D. Automatic Pressure Relief
Subsystems

1. Except as specified in 3.5.D.2 and 3.5.D.3 below, the Automatic Pressure Relief Subsystem shall be operable whenever the reactor pressure is greater than 90 psig and irradiated fuel is in the reactor vessel.
2. From and after the date that one of the five relief valves of the automatic pressure relief subsystem is made or found to be inoperable.

4.5 SURVEILLANCE REQUIREMENT
(Cont'd.)

D. Surveillance of the
Automatic Pressure Relief
Subsystem shall be
performed as follows:

1. During each operating cycle the following shall be performed:
 - a. A simulated automatic initiation which opens all pilot valves, and
 - b. With the reactor at pressure each relief valve shall be manually opened. Relief valve opening shall be verified by a compensating turbine bypass valve or control valve closure.
 - c. A logic system functional test shall be performed each refueling outage.
2. Whenever HPCI is required to be operable in accordance with 3.5.D.2, HPCI shall be tested to demonstrate operability immediately.

3.5 LIMITING CONDITION FOR OPERATION
(Cont'd.)

4.5 SURVEILLANCE REQUIREMENT
(Cont'd.)

reactor operation is permissible only during the succeeding seven (7) days provided that during such time the HPCI subsystem is operable. If the following MAPLHGR reduction factors (multipliers) are applied to Figure 3.5-1, the Automatic Pressure Relief Subsystem of ECCS shall be considered operable:
1) 0.89 for 8x8 fuel, or
2) 0.76 for 9x9 fuel.

3. From and after the date that two relief valves are found or made to be inoperable, reactor operation is permissible only during the succeeding seven days provided that during such time the HPCI subsystem is operable and the multipliers specified in 3.5.D.2. are applied.
4. If the requirements of 3.5.D.1 cannot be met, an orderly shutdown shall be initiated and the reactor pressure shall be reduced to below 90 psig within 24 hours.

3. Whenever HPCI is required to be operable in accordance with 3.5.D.3, HPCI shall be tested to demonstrate operability immediately.

E. Isolation Condenser System

- E. Surveillance of the Isolation Condenser System shall be performed as follows: