

Vogtle PEmails

From: Kallan, Paul
Sent: Wednesday, July 19, 2017 2:01 PM
To: Vogtle PEmails
Subject: Vogtle LAR 17-020 - Add Reactor Trip Signal on Safeguards Actuation Signal Time Delay and Other Miscellaneous Technical Specification Changes
Attachments: Acceptance letter for LAR 17-020(1).docx

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Subject: Vogtle LAR 17-020 - Add Reactor Trip Signal on Safeguards Actuation Signal
Time Delay and Other Miscellaneous Technical Specification Changes
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Created By: Paul.Kallan@nrc.gov

Recipients:
"Vogtle PEmails" <Vogtle.PEmails@nrc.gov>
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Recipients Received:

Mr. Brian H. Whitley, Director
Regulatory Affairs
Southern Nuclear Operating Company, Inc.
42 Inverness Center Parkway
BIN B237
Birmingham, AL 35242

SUBJECT: ACCEPTANCE REVIEW OF SOUTHERN NUCLEAR OPERATING COMPANY'S REQUEST FOR LICENSE AMENDMENT (LAR 17-020) FOR THE VOGTLE ELECTRIC GENERATING PLANT UNITS 3 AND 4 – ADD REACTOR TRIP SIGNAL ON SAFEGUARDS ACTUATION SIGNAL TIME DELAY AND OTHER MISCELLANEOUS TECHNICAL SPECIFICATION CHANGES (CAC NO. RP9629)

Dear Mr. Whitley:

By letter dated June 23, 2017 (Agencywide Documents Access and Management System (ADAMS) under Accession No. ML17179A171), Southern Nuclear Operating Company (SNC/licensee) submitted a license amendment request (LAR) 17-020 for Combined License (COL) Numbers NPF-91 and NPF-92, for the Vogtle Electric Generating Plant (VEGP) Units 3 and 4, respectively. The requested amendment requires changes to the Updated Final Safety Analysis Report (UFSAR) in the form of departures from the Plant-Specific Design Control Document (DCD) Tier 2 information and involves changes to the VEGP Units 3 and 4 COL Appendix A, Technical Specifications (TS). Specifically, the proposed changes revise plant-specific Tier 2 information to add the time delay assumed in the safety analysis for the reactor trip on a safeguards actuation ("S") signal to UFSAR Table 15.0-4a. This is also reflected in the proposed revision to TS 3.3.4, Reactor Trip System (RTS) Engineered Safety Feature Actuation System (ESFAS) Instrumentation, to add a surveillance requirement to verify the RTS response time for this "S" signal. The request also includes proposed changes to TS 3.3.7, RTS Trip Actuation Devices, to clarify that the requirements for reactor trip breaker (RTB) undervoltage and shunt trip mechanisms apply only to in-service RTBs. In addition, the request includes proposed changes to TS 3.3.9, ESFAS Manual Initiation, to correct the nonmenclature for the Chemical and Volume Control System, which is advertently stated as the Chemical Volume and Control System.

The purpose of this letter is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of the LAR. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical reviews. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Section 50.90 of Title 10 of the *Code of Federal Regulations* (10 CFR), an amendment to the license must fully describe the changes requested, and follow as far as applicable, the form prescribed for original applications. Section 52.79 of the 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff has reviewed your application and concluded that it does provide technical information in sufficient detail to enable the NRC staff to complete its detailed technical review and make an independent assessment regarding the acceptability of the proposed amendment in terms of regulatory requirements and the protection of public health and safety and the environment.

Given the lesser scope and depth of the acceptance review as compared to the detailed technical review, there may be instances in which issues that impact the NRC staff's ability to complete the detailed technical review are identified despite completion of an adequate acceptance review. You will be advised of any further information needed to support the NRC staff's detailed technical review by separate correspondence.

You have requested an issuance date of April 30, 2018. We will attempt to support that date but it will depend on whether supplements to your request are needed. If the change is needed sooner than that date to support construction, please consider submitting a request for a PAR.

If you have any questions, please contact me at (301) 415-2809 or Paul.Kallan@nrc.gov.

Sincerely,

/RA/

Paul Kallan, Senior Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket Nos.: 52-025
52-026

cc: See next page

Consistent with Section 50.90 of Title 10 of the *Code of Federal Regulations* (10 CFR), an amendment to the license must fully describe the changes requested, and follow as far as applicable, the form prescribed for original applications. Section 52.79 of the 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

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Sincerely,

/RA/

Paul Kallan, Senior Project Manager
 Licensing Branch 4
 Division of New Reactor Licensing
 Office of New Reactors

Docket Nos.: 52-025
 52-026

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(Revised 05/30/2017)

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