

**LICENSEE EVENT REPORT (LER)**

FACILITY NAME (1) Dresden Nuclear Power Station, Unit 3 DOCKET NUMBER (2) 050002491 OF 3 PAGE (3)

TITLE (4) Fuel Pool Area Radiation Monitor RM 1705-16A Setpoint Exceeded  
Technical Specification Limit Due to Instrument Setpoint Drift

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		
04	20	87	87	004	00	05	14	87	N/A		
									DOCKET NUMBER(S)		
									05000		

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)

OPERATING MODE (9) N	20.402(b)	20.408(a)	80.73a)(2)(iv)	73.71(b)
POWER LEVEL (10) 001	20.408(a)(1)(i)	80.38(a)(1)	80.73a)(2)(v)	73.71(a)
	20.408(a)(1)(ii)	80.38(a)(2)	80.73a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 305A)
	20.408(a)(1)(iii)	X 80.73a)(2)(i)	80.73a)(2)(vii)(A)	
	20.408(a)(1)(iv)	80.73a)(2)(ii)	80.73a)(2)(vii)(B)	
	20.408(a)(1)(v)	80.73a)(2)(iii)	80.73a)(2)(viii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME S. Merritt Telephone Number 815 942-2920  
Technical Staff Engineer (X-421) AREA CODE 815 TELEPHONE NUMBER 942-2920

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPSDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPSDS
X	I/L	MON	G080	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 20, 1987 while Unit 3 was in the startup mode at 1% power at 0830 hours, it was found that the upscale trip setpoint for the refuel floor fuel pool Area Radiation Monitor 1705-16A exceeded Technical Specification 3.2.D.3 limits by the Instrument Maintenance Department while performing a quarterly surveillance (Dresden Instrument Surveillance 1700-15, Refuel Floor Radiation Monitor Calibration and Functional Test). The as-found upscale trip setpoint was 200 mR/hr. The Technical Specification limit is less than or equal to 100 mR/hr. The setpoint for the monitor was readjusted and verified operable at 90 mR/hr. The cause of the event was attributed to instrument setpoint drift. Safety significance of the event was minimal; Refuel floor fuel pool Area Radiation Monitor 1705-16B was fully operational with an upscale trip setpoint of 85 mR/hr. A previous occurrence involving a refuel floor ARM failure is recorded on Reportable Occurrence 86-008 on Docket #050237.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 7	- 0 0 4	- 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 388A's) (17)

PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor - 2527 MWt rated core thermal power. Energy Industry Identification System (EIIS) codes are identified in the text as [XX].

EVENT IDENTIFICATION:

Fuel Pool Area Radiation Monitor RM 1705-16A setpoint found greater than Technical Specification limit due to instrument setpoint drift.

A. CONDITIONS PRIOR TO EVENT:

Unit: 3                      Event Date: April 20, 1987                      Event Time; 0830 hours  
 Reactor Mode: N      Mode Name: Startup                      Power Level: 01%  
 Reactor Pressure: 493 psig

This report was initiated by Deviation Report #12-3-87-34.

B. DESCRIPTION OF EVENT:

On April 20, 1987 while Unit 3 was in the startup mode at 01% power at 0830 hours, the upscale trip setpoint for the refuel floor fuel pool Area Radiation Monitor (ARM) [IL] 1705-16A was found by the Instrument Maintenance Department (IMD) to be greater than the setpoint specified in Technical Specification (T.S.) 3.2.D.3. According to T.S. 3.2.D.3 the trip setting for the refueling floor radiation monitors shall be set at less than or equal to 100 mR/hr.

On 4/20/87 at 0808 hours an IMD Technician began performing Dresden Instrument Surveillance (DIS) 1700-15. DIS 1700-15, Refuel Floor Radiation Monitor Calibration and Functional Test, is performed on a quarterly basis per T.S. Table 4.2.1. At 0830 hours, while checking the setpoint, he discovered that the upscale trip setpoint for RM 1705-16A was 200 mR/hr, which is greater than the T.S. limit of less than or equal to 100 mR/hr. The normal acceptable range is 80 to 90 mR/hr. The IMD Technician readjusted the setpoint to 90 mR/hr per DIS 1700-15 criteria. The surveillance was completed at 1037 hours on 4/20/87.

C. APPARENT CAUSE OF EVENT:

The cause of the fuel pool Area Radiation Monitor RM 1705-16A upscale trip setpoint being greater than the T.S. limit is instrument setpoint drift. This was determined by performing an investigation in accordance with Dresden Instrument Procedure (DIP) 010-9, Instrument Maintenance Department Setpoint Drift Work Sheet.

D. SAFETY ANALYSIS OF EVENT:

The safety significance of the event is considered minimal because the redundant refuel floor fuel pool ARM 1705-16B was fully operational and its upscale trip setpoint was found to be at 85 mR/hr.

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TEXT (If more space is required, use additional NRC Form 388A's) (17)

**E. CORRECTIVE ACTION:**

Corrective action consisted of readjusting and functionally verifying the trip upscale setpoint of ARM 1705-16A at 90 mR/hr. Continued periodic surveillances will verify proper operation of this equipment.

**F. PREVIOUS EVENTS:**

A previous occurrence involving a refuel floor ARM failure is recorded on Reportable Occurrence 86-008 on Docket #050237. However, that occurrence involved a refuel floor ARM upscale failure due to a failure of its Geiger-Mueller tube.

**G. COMPONENT FAILURE:**

The instrument is not reportable to NPRDS.

Manufacturer: General Electric Company

Nomenclature: 194 x 927 Sensor and Converter

Part Number: 1705-16A



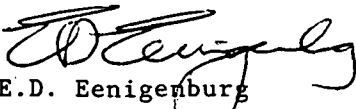
**Commonwealth Edison**  
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Telephone 815/942-2920

May 14, 1987

EDE LTR #87-316

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Licensee Event Report #87-004-0, Docket #050249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(i)(B).

  
E.D. Eenigenburg  
Station Manager  
Dresden Nuclear Power Station

EDE/kjl

Enclosure

cc: A. Bert Davis, Acting Regional Administrator, Region III  
File/NRC  
File/Numerical

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