

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Dresden Nuclear Power Station, Unit 2 DOCKET NUMBER (2) 0 5 0 0 0 2 3 7 1 OF 0 3 PAGE (3)

TITLE (4) Failure to Calculate Drywell Floor and Equipment Leakage Values on a Four Hour Interval Due to Personnel Error

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	4	2	8	7	0	1	5	0	0	N/A	0 5 0 0 0
0	4	2	8	7	0	1	5	0	0	N/A	0 5 0 0 0

OPERATING MODE (9) N

POWER LEVEL (10) 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)

20.402(b)	20.405(a)	50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.73(a)(1)	50.73(a)(2)(v)	73.71(a)
20.405(a)(1)(ii)	50.73(a)(2)	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 355A)
20.405(a)(1)(iii)	X 50.73(a)(2)(ii)	50.73(a)(2)(vii)(A)	
20.405(a)(1)(iv)	50.73(a)(2)(iii)	50.73(a)(2)(vii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iv)	50.73(a)(2)(vii)(C)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: Anthony Anandappa, Technical Staff Engineer (X-548)

TELEPHONE NUMBER: 8 1 5 9 4 2 1 - 2 9 2 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 26, 1987 while startup operations were in progress in accordance with Dresden General Procedure (DGP) 1-1, the drywell floor and equipment drain sumps were not pumped at 1600 hours as required by Technical Specification 4.6.D.1. The sumps were subsequently pumped at 1712 hours. The floor drain leakage was 0 gpm and equipment drain leakage was 0.449 gpm. At 1200 hours that day, when the sumps were previously pumped, the recorded leakage was 0 gpm and 0.375 gpm for the floor drain and equipment drain sumps respectively. The event is of minimal safety significance as the leakage rates were below the Technical Specification 3.6.D.1 limits. The cause of the event was due to personnel error. The Division Vice President and Station Manager arrived in the Control Room within hours of the event and interviewed the Operators involved, and reinforced to them the requirement to pump the sumps on time. The last similar event was reported in Reportable Occurrence #86-006-0, under Docket #050237.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8   7	-   0   1   5	-   0   0	0   2	OF	0   3

TEXT (If more space is required, use additional NRC Form 368A's) (17)

PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor - 2527 MWt rated core thermal power. Energy Industry Identification System (EIIS) codes are identified in the text as [XX].

EVENT IDENTIFICATION:

Failure to Calculate Drywell Floor and Equipment Drain Leakage Values on a Four Hour Interval Due to Personnel Error.

A. CONDITIONS PRIOR TO EVENT:

Reactor Mode: N - Startup                      Reactor Pressure: 90 psig  
Reactor Power: 0.1%

B. DESCRIPTION OF EVENT:

During startup operations at 0.1% rated thermal power in accordance with Dresden General Procedure (DGP) 1-1, the unit Nuclear Station Operator (NSO) failed to pump the drywell floor and equipment drain sumps [WK] at 1600 hours after receiving the time triggered operator selected alarm. Subsequently, the Station Control Room Engineer (SCRE), while performing his rounds, observed that a scheduled entry in the NSO log book at 1600 hours on drywell floor and equipment drain leakage was not made as required by Technical Specification 4.6.D.1. Upon questioning the Operators it was determined that the requirement was missed due to the startup activities. All startup activity was immediately suspended and the sumps were pumped at 1712 hours. A floor drain leakage of 0 gpm and Equipment drain leakage of 0.449 gpm were then recorded. An immediate investigation was conducted by the Nuclear Stations Division Vice President and Station Manager to determine the cause of the error and corrective action. Startup activity was then resumed. The normal four hour containment [IJ] leakage surveillance schedule was resumed at 2000 hours.

C. CAUSE OF EVENT:

The cause of the event was due to personnel error. The time triggered operator selected sump pumping reminder alarm sounded, and the alarm was acknowledged. However, due to the startup evolutions occurring at the time, the NSO failed to pump the sumps in a timely manner. At the time of this event two NSO's were manning the main Control Room panels; one was primarily involved with pulling control rods and monitoring reactor conditions, while the other was assisting with balance of plant operations associated with the startup. Although neither NSO could recall receiving the alarm or acknowledging it, the alarm typer showed that the alarm was received. The alarm in question is received on a video monitor on the Operator's console and does not result in an illuminated annunciator window. An unrelated alarm was received three seconds later and it is believed that the Operator acknowledged both alarms while only recognizing the last one received. The root cause of this event was attributed to personnel error on the part of the NSO. This report is submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(i)(B).

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		8   7	—   0   1   5	—   0   0	0   3	OF	0   3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

D. SAFETY ANALYSIS:

The basis of the four hour interval requirement for pumping the drywell floor drain and equipment drain sumps is to calculate primary system leakage rates within the primary containment in a timely manner. Thus, any significant loss of integrity of components within the primary containment would be detected promptly. In this event, reactor power was only 0.1% rated thermal and reactor pressure was only 90 psig. This would have mitigated the consequences of any significant primary system leakage. Any such leakage would have been contained within the primary containment structure [NH]. The NSO failed to record and calculate the required values at 1600 hours; however, this error was discovered promptly by a Shift Supervisor and the requirement was completed one hour and 12 minutes late.

The leakage into the drywell floor drain sump was 0 gpm, which complied with the Technical Specification 3.6.D.1 limitation of 5 gpm for leakage within the primary containment from unidentified sources. The recorded value for leakage into the drywell equipment drain sump, which is representative of leakage within the primary containment from identified sources, was 0.449 gpm. Thus, the total leakage within the primary containment, at 0.449 gpm, was well below the Technical Specification 3.6.D.1 limit of 25 gpm. Also, since no unidentified leakage was observed, no further investigation or entry within the primary containment was required by Technical Specification 4.6.D.2. Any significant leakage within the primary containment would have resulted in annunciation of sump high level alarms. For these reasons, the safety significance of this event was minimal.

E. CORRECTIVE ACTIONS:

The Division Vice President and Station Manager arrived in the Control Room within hours of the event and interviewed the personnel involved. As a corrective action the Division Vice President and Station Manager reinforced to the individuals on the Technical Specification requirement to pump the sumps on time. No further action was deemed necessary, although further consideration is being given to a longer term corrective action which would provide a redundant, diverse operator aid in order to facilitate Operator recognition of this requirement.

F. PREVIOUS OCCURRENCES:

The last similar event was reported in Reportable Occurrence 86-006-0 under Docket #050237.

G. COMPONENT FAILURE DATA:

There were no component failures in this event. Therefore, component failure data is not applicable.



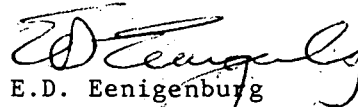
**Commonwealth Edison**  
Dresden Nuclear Power Station  
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Telephone 815/942-2920

May 14, 1987

EDE LTR #87-319

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Licensee Event Report #87-015-0, Docket #050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(i)(B).

  
E.D. Eenigenburg  
Station Manager  
Dresden Nuclear Power Station

EDE/kjl

Enclosure

cc: A. Bert Davis, Regional Administrator, Region III  
File/NRC  
File/Numerical

IE22  
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