



Commonwealth Edison
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Address Reply to: Post Office Box 767
Chicago, Illinois 60690

September 21, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Dresden Station Units 2 and 3
Proposed Amendments to Technical
Specification for Operating Licenses
DPR-19 and DPR-25, Installation of
Bypass Valve Around MOV 2-1201-1 in
the Reactor Water Cleanup System
NRC Docket No. 50-237 and 50-249

References (a): B. Rybak letter to H. R. Denton dated
January 13, 1984.

(b): B. Rybak letter to H. R. Denton dated
May 1, 1984.

Dear Mr. Denton:

By this letter, pursuant to 10 CFR 50.59, Commonwealth Edison proposes to amend Provisional Operating License DPR-19 for Dresden Unit 2 and to supplement the amendment request for Unit 3 which was submitted per reference (a) and supplemented earlier by reference (b). The proposed change for Unit 2 is to revise Table 3.7.1 which lists the primary containment isolation valves. The change would increase the number of inboard isolation valves on the Reactor Cleanup (RWCU) System from 1 to 2. This request results from the planned addition of a small diameter (2 inch) bypass line around the eight (8) inch inboard isolation valve 1201-1 during the refueling outage scheduled to start in October. The proposed bypass line will contain a motor-operated valve (2-1201-1A) which is required to have a containment isolation function and therefore must be added to Table 3.7.1. A similar modification was made on Unit 3 earlier this year with no operational problems experienced to date.

The bypass line and valve around the inboard containment isolation valve M.O. 2-1201-1 in the RWCU System of Unit 2 will be installed to alleviate potential RWCU System transients that have occurred in the past on its sister unit (Unit 3). That potential transient would be a water hammer that occurs between the inboard isolation valve and the primary containment penetration and would result from the opening of the inboard isolation valve (M.O. 2-1201-1) when the reactor is pressurized, and the RWCU System at a lower pressure such as during unit startup.

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The bypass line would have a 2-inch globe valve which will be opened to pressure the RWCU System to the pressure control valve PCV-1217 prior to the RWCU System startup. Once the RWCU System has been pressurized, inboard isolation valve (M.O. 2-1201-1) can be opened without inducing significant loads on the piping and supports. The bypass valve (M.O. 2-1201-1A) will then be closed for normal RWCU System operation. The bypass valve will isolate on a Group III isolation signal.

As for Unit 3, our previous submittals shows the normal position of 3-1201-1A on Table 3.7.1 as open. Dresden considers the proper position is closed during normal operations. Therefore, we have revised Table 3.7.1 to show the valve as normally closed.

The proposed changes to Table 3.7.1 for Units 2 and 3 are enclosed in Attachment 1. The attached change has received both On-site and Off-site review and approval. We have reviewed this amendment request and find that no significant hazards consideration exists. Our review is documented in Attachment 2. Commonwealth Edison is notifying the State of Illinois of our request of this amendment by transmittal of a copy of this letter and its attachment to the designated State Official.

In accordance with 10 CFR 170, a fee remittance in the amount of \$150.00 is enclosed.

Three (3) signed originals and thirty-seven (37) copies of this transmittal and its attachments are provided for your use.

Very truly yours,

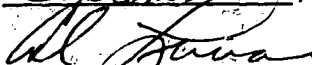

B. Rybak

Nuclear Licensing Administrator

lm

cc: Region III Inspector - Dresden
R. Gilbert - NRR
G. Wright - Ill.

SUBSCRIBED AND SWORN to
before me this 21st day
of September, 1984


Notary Public

Attachments (1): Technical Specification Change to DPR-19 and 25
(2): Evaluation of Significant Hazards Consideration