



Commonwealth Edison
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Address Reply to: Post Office Box 767
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March 5, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Dresden Station Units 2 and 3
Proposed Amendment to Technical
Specification for Facility Operating
Licenses DPR-19 and 25
Changes to Table 3.7.1,
Containment Isolation Valves
NRC Docket Nos. 50-237 and 249

Reference (a): D. M. Crutchfield letter to D. L. Farrar
dated October 5, 1983.

Dear Mr. Denton:

Pursuant to 10 CFR 50.59, Commonwealth Edison proposes to amend Appendix A, Technical Specification, to Facility Operating License Nos. DPR-19 and 25 for Dresden Station Units 2 and 3 respectively. The referenced letter requested changes to referenced letter 3.7.1 requiring as a minimum addition of four LPCI suction valves and two torus spray suction valves to that table. We were also asked to review whether any other valves should also be included on Table 3.7.1 and requested to include those valves in any amendment submittal.

The following primary containment isolation valves have been added to Table 3.7.1:

LPCI Suction Valves 1501-5A, B, C, D
LPCI Torus Spray Valves 1501-18A, B, 19A, B
LPCI Test Return Valves 1501-38A, B, 20A, B
LPCI Injection Valves 1501-25A, B, 22A, B
LPCI Drywell Spray Valves 1501-27A, B, 28A, B

Core Spray Suction Valves 1402-3A, B,
Core Injection Valves 1402-24A, B, 25A, B
Core Spray Test Return Valves 1402-4A, B

HPCI Suction Valves 2301-35, 36

Reactor Head Spray Valve 205-2-4 has been changed from Group III to Group II to reflect the actual plant conditions. Other corrections to reflect actual operating conditions were also made. These changes are as follows:

8403130410 840305
PDR ADCK 05000237
P PDR

Pool
w/ check \$ 4,400
201169

1. Recirculation Loop Sample Valves 220-44 and 45 and the Drywell Floor Drain and Drywell Equipment Valves 2001-105, 2001-106 and 2001-5 and 6, respectively are now shown normally closed and stay closed on the initiating signal.
2. The number of valves in the Drywell Air Sampling System and Shutdown Cooling was revised to show the actual number of primary containment valves.

The proposed change to Table 3.7.1 is enclosed in Attachment 1. The attached change has been received both Onsite and Offsite review and approval. We have reviewed this amendment request and find that no significant hazards consideration exists. Our review is documented in Attachment 2. Commonwealth Edison is notifying the State of Illinois of our request of this amendment by transmittal of a copy of this letter and its attachments to the designated State Official.

We have determined that this request is a 10 CFR 170, Class I and Class II amendment request. As such a fee remittance of \$4,400.00 is enclosed.

Please direct any questions you may have concerning this matter to this office.

Three (3) signed originals and thirty-seven (37) copies of this transmittal and its attachments are provided for your use.

Very truly yours,




B. Rybak
Nuclear Licensing Administrator

lm

- Attachments (1): Technical Specification Change to DPR-19 and 25
 (2): Evaluation of Significant Hazards Consideration

cc: Region III Inspector - Dresden
 R. Gilbert - NRR
 G. Wright - Ill.

SUBSCRIBED and SWORN to
 before me this 5th day
 of March, 1984


 Notary Public