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February 9, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Dresden, Quad Cities and LaSalle County Stations
Response to Generic Letter No. 83-36 "NUREG-0737
Technical Specifications"
NRC Docket Nos. 50-237/249, 254/265 and 373/374

Reference (a): D. G. Eisenhut letter to All Pressurized
Water Reactor Licenses dated November 1,
1983 (NL-83-0506)

Dear Mr. Denton:

Reference (a) requested that licensees submit Technical Specifications or a submittal schedule for NUREG-0737 items scheduled after December 31, 1981.

At Dresden and Quad Cities Stations a number of the TMI items are not yet operational and will not be completed until mid-1984. In addition, some of the subject TMI items require revisions to sections of the Technical Specifications which are currently in the review process under the Radiological Effluent Technical Specifications (RETS) program. Proposing changes at this time will only confuse the issue. It is our position that some of these Technical Specification changes be considered after the RETS package is issued by the NRC.

To avoid piecemeal submittals and assuming completion of the RETS review we will submit, by August 15, 1984, all appropriate Technical Specification amendment requests.

LaSalle County's Technical Specification contains all applicable specifications described in the subject letter.

The attachment to this letter provides the status and schedule of those items addressed in Reference (a).

To the best of my knowledge and belief the statements contained in the Attachment are true and correct. In some respects, these statements are not based on my personal knowledge but upon information furnished by other Commonwealth Edison employees, consultants, and contractors. Such information has been reviewed in accordance with Company practice and I believe it to be reliable.

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Please address any questions that you or your staff may have concerning our response to Generic Letter No. 83-36 to this office.

Respectfully,

P. L. Barnes

P. L. Barnes
Nuclear Licensing Administrator

Attachment

cc: J. G. Keppler - RIII
R. Bevan - ORB 2
A. Bournia - LB 2
R. Gilbert - ORB 5
RIII Inspector - D/QC/LSC
NRC Document Control Desk

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Attachment 1

Dresden Response to
Generic Letter 83-36 "NUREG-0737
Technical Specifications"
NRC Docket Nos. 50-237/249

8113N

Dresden Station

1) Reactor Coolant System Vents (II.B.1)

This item is not applicable to Dresden Station since we have a turbine driven High Pressure Coolant Injection System along with the Isolation Condenser System. Therefore, no Technical Specification changes are required.

2) Postaccident Sampling (II.B.3)

Postaccident sampling requirements are covered in the draft RETS amendment.

3) Noble Gas Effluent Monitors (II.F.1.1)

The Noble Gas Effluent Monitors and appropriate specifications is included in the draft RETS amendment.

4) Sampling and Analysis of Plant Effluents (II.F.1.2)

The Sampling and Analysis of Plant Effluents Program is covered in the draft RETS amendment.

5) Containment High-Range Radiation Monitor (II.F.1.3)

The monitors will be included in the postaccident monitoring instrumentation table which are proposed as a result of the Generic Letter. Operability and surveillance requirements are specified based on the guidance provided in Enclosure 3 and specifications for similar instrumentation presently included in the Technical Specifications.

6) Containment Pressure Monitor (II.F.1.4)

The monitors will be included in the postaccident monitoring instrumentation tables to be proposed as a result of the Generic Letter. Operability and surveillance requirements are specified based on guidance provided in Enclosure 3 and specifications for containment pressure monitors presently included in the Technical Specifications.

7) Containment Water Level Monitor (II.F.1.5)

The montors will be included in the postaccident monitoring instrumentation tables to be proposed as a result of the Generic Letter. Operability and surveillance requirements will be specified based on the guidance provided in Enclosure 3 and specifcation for suppression pool level instrumentation presntly included in the Technical Specifications.

8) Containment Hydrogen Monitor (II.F.1.6)

The installation and testing of this system has not been completed. It is scheduled to be in service in April, 1984. Technical Specification changes will be submitted after completion of the modification.

9) Control Room Habitability Requirements (II.D.3.4)

The new Control Room HVAC system installation and testing has not been completed. It is scheduled to be in service in June, 1984. Technical Specification changes will be submitted after completion of the modification.

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Attachment 2

Quad Cities Response to
Generic Letter 83-36 "NUREG-0737
Technical Specifications"
NRC Docket Nos. 254/265

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FOR REVIEW BY BUREAU

Quad Cities Station

1. Reactor Coolant System Vents (II.B.1)

This item is not applicable to Quad Cities Station, since we do not have isolation condensers. Therefore, no Tech Spec changes are proposed.

2. Post-Accident Sampling (II.B.3.)

Appropriate words in accordance with the NRC staff guidance will be proposed to be included in the operating licenses, DPR-29 and DPR-30.

3. Noble Gas Effluent Monitors (II.F.1.1)

Although the SPING unit is operable for the reactor building ventilation exhaust, and the SPING unit for the main chimney effluent pathway has been operable, it is considered more appropriate to propose Tech Spec changes after the RETS are issued by the NRC and implemented at the Station. Proposing changes now would serve only to confuse the issue and interference with compliance with the RETS. We will submit Tech Spec changes to incorporate the SPING monitors into the RETS after issuance by the NRC.

4. Sampling and Analysis of Plant Effluents (II.F.1.2)

This system is not yet operational. It is scheduled to be in service in March, 1984. However, the Victoreen monitor system is tied into the SPING. Appropriate Tech Specs for Victoreen system will be provided after the RETS have been issued.

5. Containment High-Range Radiation Monitor (II.F.1.3)

The monitors are proposed to be included in the post-accident monitoring instrumentation tables in the Tech Specs. Operability and surveillance requirements will be provided, in accordance with the staff guidance provided in the Generic Letter.

6. Containment Pressure Monitor (II.F.1.4)

These monitors are proposed to be included in the post-accident monitoring instrumentation tables in the Tech Specs. The 0-250 psig indicators will be integrated into the operability requirements and surveillance test frequencies given for the pre-existing pressure instruments.

7. Containment Water Level Monitor (II.F.1.5)

These monitors are proposed to be included in the post-accident monitoring instrumentation tables in the Tech Specs. The 0-30 ft. indicator will be integrated into the existing Tech Spec operability requirements in a fashion so as to provide for the operability of both narrow and wide range torus level instruments. Appropriate surveillance requirements will also provide.

8. Containment Hydrogen Monitor (II.F.1.6.)

This system is not yet operational. It is schedule to be in service in April, 1984. Tech Spec change will be submitted after the system is operational.

9. Control Room Habitability Requirements (II.D.3.4.)

The new Control Room HVAC system is not yet operational. It scheduled to be in service in July, 1984. Tech Spec changes will be submitted after the system is operational.

Attachment 3

LaSalle County Station Response to Generic Letter
No. 83-36 "NUREG-0737 Technical Specifications"
NRC Docket Nos. 50-373/374

The following is an item by item response to the issues addressed in Enclosure 1 to reference (a).

- (1) LSCS does not have an isolation condenser; therefore, this item is not applicable.

Items (2) through (9) are currently in the Unit 1 and the Unit 2 Technical Specifications:

- (2) Section 6.2.F.3
- (3) Table 4.3.7.11-1
- (4) Section 6.2.F.2
- (5) Table 3.3.7.5-1
- (6) Table 3.3.7.5-1
- (7) Table 3.3.7.5-1
- (8) Table 3.3.7.5-1
- (9) Reference 3/4.3.7.1
3/4.3.7.8
3/4.7.2
Table 3.7.7-1

No Technical Specification changes are required.