



Commonwealth Edison
One First National Plaza, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

November 15, 1983

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Dresden Station Units 2 and 3
Quad Cities Station Units 1 and 2
Zion Station Units 1 and 2
LaSalle County Station Units 1 and 2
Byron Station Units 1 and 2
Braidwood Station Units 1 and 2
Revised Commitment Regarding NUREG-0737
Supplement 1, Generic Letter No. 82-33
NRC Docket Nos. 50-237/249, 50-254/265,
50-295/304, 50-373/374, 50-454/455,
and 50-456/457

- References (a): D. G. Eisenhut letter to All OLs, Applicants
and CPs dated December 17, 1982 (Generic
Letter No. 82-33)
- (b): Cordell Reed letter to H. R. Denton
dated April 14, 1983.
- (c): E. D. Swartz letter to H. R. Denton
dated July 20, 1983.
- (d): Cordell Reed letter to H. R. Denton
dated July 28, 1983.
- (e): Cordell Reed letter to H. R. Denton
dated August 25, 1983.

Dear Mr. Denton:

In our letter of August 25, 1983, Reference (e), we submitted a proposed schedule for completion of work required by Generic Letter 82-33. In a telecon meeting on August 31, 1983 with D. M. Crutchfield and W. A. Paulson of your staff, we discussed certain adjustments to the Reference (e) schedule. As a result of that telecon meeting and subsequent conversations, CECo is submitting a final, agreed upon schedule in the form of Attachment A to this letter. It is our understanding that these dates will appear in the form of Orders for our OL's and as License Conditions for our the NTOL's.

A question was raised as to whether the DCRDR schedule can be advanced for the plants that are further out in our review process in order to take advantage of the "learning curve" that should result. It is our intent to proceed in a serial and continuous manner in our DCRDR program. If the experience gained in previous reviews benefits

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succeeding reviews in a way that allows a reduction in the length of the total review process, we will take advantage of that experience and advance our DCRDR program schedule as appropriate.

It should be noted that for our Zion Station, the following significant improvements have already been made in the areas of control room human factors and accident monitoring instrumentation:

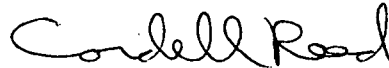
- (1) In accordance with Item E.2 of the Zion Confirmatory Order of February 29, 1980, Commonwealth Edison performed a review of control room displays for the purpose of identifying improvements which could increase the operator's ability to assess plant conditions.
- (2) In 1981, Zion Station adopted Standard Technical Specifications for accident monitoring instrumentation, which are based on Regulatory Guide 1.97.
- (3) Zion Station has completed the installation of the accident monitoring instrumentation of NUREG-0737 Items II.E.1.1, 2, 3, 4, 5, and 6.

To the best of my knowledge and belief, the statements contained herein and in the Attachments are true and correct. In some respects these statements are not based on my personal knowledge but upon information furnished by other Commonwealth Edison employees and Consultants. Such information has been reviewed in accordance with Company practice and I believe it to be reliable.

Please address any further questions that you or your Staff may have concerning this submittal to the Director of Nuclear Licensing.

One (1) signed original and forty (40) copies of this letter with Attachments are provided for your use.

Very truly yours,



Cordell Reed
Vice-President
Nuclear Operations

BR/lm

Attachments

cc: Mr. J. G. Keppler, Region III
RIII Inspectors - D/QC/Z/LS/B/B

ATTACHMENT A

Revised Section 9 Schedule to the Cordell Reed
Letter to H.R. Denton dated April 14, 1983

7230N

SECTION 9 SCHEDULES

EMERGENCY RESPONSE CAPABILITY

DRESDEN

QUAD CITIES

LASALLE

ZION

BYRON

BRAIDWOOD

I. SAFETY PARAMETER DISPLAY SYSTEM (SPDS)

a. Submit Safety Analysis

- | | | | | | | |
|---|----------|----------|----------|----------|----------|----------|
| 1) Criteria for parameter selection including V&V description | 12-31-83 | 12-31-83 | 12-31-83 | 12-31-83 | 12-31-83 | 12-31-83 |
| 2) HFR of data display and functions (assumes NRC acceptance of DCRDR final summary report within 2 months) | 01-01-86 | 01-01-86 | 07-01-86 | 01-01-87 | 08-01-87 | 08-01-87 |
| 3) Verify parameter selection (assumes NRC acceptance of DCRDR final summary report within 2 months) | 01-01-86 | 01-01-86 | 07-01-86 | 01-01-87 | 08-01-87 | 08-01-87 |

b. SPDS Operational - Defined as design, hardware and software installation, functional testing, and initial operator training complete

01-01-85	06-01-85	Unit 1 complete Unit 2 prior to completion of startup test program	Complete	Prior to commercial service	Prior to commercial service
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c. Modifications to SPDS resulting from DCRDR, SPDS HFR, or RG 1.97

- | | | | | | | |
|--------------------|----------|----------|----------|----------|----------|----------|
| 1) Submit schedule | 01-01-86 | 01-01-86 | 07-01-86 | 01-01-87 | 08-01-87 | 08-01-87 |
|--------------------|----------|----------|----------|----------|----------|----------|

II. DETAILED CONTROL ROOM DESIGN REVIEW (DCRDR)

- | | | | | | | |
|--------------------------------|----------|----------|----------|----------|----------|----------|
| a. Submit Program Plan | Complete | Complete | Complete | Complete | Complete | Complete |
| b. Submit Final Summary Report | 05-01-85 | 05-01-85 | 11-01-85 | 05-01-86 | 12-01-86 | 12-01-86 |

III. REGULATORY GUIDE 1.97 - REVISION 2

- | | | | | | | |
|--|----------|----------|----------|----------|----------|----------|
| a. Submit a preliminary report describing how the requirements of Reg. Guide 1.97 specified in Supplement 1 to NUREG-0737 have been or will be met. | 08-01-85 | 08-01-85 | Complete | 08-01-86 | 03-01-87 | 03-01-87 |
| b. Submit a final report including a schedule for installation (assumes NRC acceptance of DCRDR results within 3 months of submittal date of DCRDR summary report for that Station). | 02-01-86 | 02-01-86 | 08-01-86 | 02-01-87 | 09-01-87 | 09-01-87 |

EMERGENCY RESPONSE CAPABILITY

DRESDEN QUAD CITIES LASALLE ZION BYRON BRAIDWOOD

IV. UPGRADE EMERGENCY OPERATING PROCEDURES (EOPs)

a.	Submit Procedures Generation Package assuming NRC approval of BWR OG EPGs Revision 3 by 09-30-83 and NRC approval of W OG EPGs Revision 1 by 3-1-84.	09-30-84	09-30-84	09-30-84	12-31-84	03-01-85	03-01-85
b.	Implement upgraded EOPs based upon Revision 3 for BWRs and Revision 1 for PWRs	09-30-85	09-30-85	09-30-85	12-31-85	03-01-86	03-01-86

V. EMERGENCY RESPONSE FACILITIES

a.	Technical Support Center (TSC)						
	1) Complete with the exception of impact due to RG 1.97 and HFR	Complete	Complete	Complete	Complete	Prior to Fuel Load	Prior to Fuel Load
	2) Modifications to TSC resulting from RG 1.97 and HFR (assumes NRC acceptance of DCRDR results with 3 months of submittal date of DCRDR summary report for that station)						
	a. Submit schedule	11-01-85	11-01-85	05-01-86	11-01-86	06-01-87	06-01-87
b.	Operational Support Center (OSC)	Complete	Complete	Complete	Complete	Prior to Fuel Load	Prior to Fuel Load
c.	Emergency Operations Facility (EOF)						
	1) Complete with the exception of impact due to RG 1.97 and HFR, and A-model, and industrial security	Complete	Complete	Complete	Complete	Prior to Fuel Load	Prior to Fuel Load
	a. Modifications to EOF resulting from RG 1.97 and HFR						
	1) Submit schedule	11-01-85	11-01-85	05-01-86	11-01-86	06-01-87	06-01-87
	b. Implement A-model	01-01-85	01-01-85	Complete	Complete	Prior to Fuel load	Prior to Fuel load
	c. Implement Industrial Security including remote alarm	06-01-84	06-01-84	06-01-84	Complete	Prior to Fuel Load	Prior to Fuel Load

D/QC/Z/LSC/BY/BW

NOVEMBER 18, 1983

To:

C. Reed / N.A. Kershaw
J.S. Abel
H.E. Bliss
L.J. Burke (RIII only)
T.C. Cihlar / G.E. Peterson
R. Cosaro
L.O. DelGeorge
J.D. Deress / J.T. Westermeier
G.J. Diederich
W.L. Eck (IEB, IEC, IEN only)
E.E. Fitzpatrick
D.P. Galle
K.L. Graesser / G.J. Pliml
J.F. Gudac
R.H. Holyoak
J.H. Hughes
R.F. Janecek (IEB only)
R.E. Jortberg
N.J. Kalivianakis
A.W. Kleinrath
D.E. Lindvall
J.J. Maley / B.R. Shelton
T.E. Quaka
R.E. Querio
V.I. Schlosser / C.J. Tomashek
D.J. Scott
D.L. Shamblin
W.J. Shewski
N.P. Smith
T.R. Sommerfield
J. Gallo - IL&B (Washington)

K.J. Hansing
W.L. Stiede
G.P. Wagner
T.E. Watts
M.J. Wallace
E.R. Zebus (IEB only)
Nuclear Safety Group - LaSalle
QA Engineer - Braidwood
QA Engineer - Byron
QA Engineer - Dresden
QA Engineer - Quad Cities
QA Engineer - LaSalle
QA Engineer - Zion
S. Woodford - IL&B
P.P. Steptoe - IL&B (Ltrs only)
J.W. Johnson - W/Z
W.E. Kortier - W
M.E. Bray / J. Nash - G.E.
M.A. Bowidowicz - S&L
G.F. Owsley - Exxon
G. Wright - State of Illinois
- (NRC/CECo Ltrs only)
W.R. Bird - Consumers Power Co.
- (RIII only)
S.J. Brewer - Marble Hill
- (CECo Ltrs only)
K.H. Shafer - I-I
R. Goldsmith - CBE
- (CECo Ltrs only)
NL Distribution
H.E. Pearson - Quadrex

In the judgement of the Nuclear Licensing Administrator, the attached document contains information that may be useful to you or your organization. No specific action or response by Commonwealth Edison is required at this time.

IDENTIFICATION OF ATTACHED DOCUMENT:

All Stations - November 4, 1983 Federal Register Notice; NRC Final Rule incorporating into regulations by reference the Summer 1982 Addenda of ASME Code.

PL Barnes
NL-83-0551

FILE: Rulemakings

meeting, and the amendment relieves restrictions on the handling of lemons. It is necessary to effectuate the declared purposes of the Act to make these regulatory provisions effective as specified, and handlers have been apprised of such provisions and the effective time.

List of Subjects in 7 CFR Part 910:

Marketing agreements and orders, California, Arizona, Lemons.

PART 910—[AMENDED]

1. Section 910.738 is added as follows:

§ 910.738 Lemons Regulation 436.

The quantity of lemons grown in California and Arizona which may be handled during the period November 6, 1983, through November 12, 1983, is established at 225,000 cartons.

2. Section 910.735 Lemon Regulation 435 (48 FR 49837) is revised to read as follows:

§ 910.735 Lemon Regulation 435.

The quantity of lemons grown in California and Arizona which may be handled during the period October 30, 1983, through November 5, 1983 is established at 220,000 cartons.

(Secs. 1-19, 48 Stat. 31, as amended; 7 U.S.C. 601-674)

Dated: November 3, 1983.

Charles R. Brader,

Director, Fruit and Vegetable Division,
Agricultural Marketing Service.

FR Doc. 83-30132 Filed 11-3-83; 11:25 am

BILLING CODE 3410-02-M

NUCLEAR REGULATORY COMMISSION

10 CFR Part 50

Codes and Standards for Nuclear Power Plants; Summer 1982 Addenda

AGENCY: Nuclear Regulatory Commission.

ACTION: Final rule.

SUMMARY: The Commission is amending its regulations to incorporate by reference the Summer 1982 Addenda of the American Society of Mechanical Engineers (ASME) Boiler Pressure Vessel Code. The sections of the ASME Code being incorporated provide rules for the construction on nuclear power plant components. Adoption of these amendments will permit the use of improved methods for construction of nuclear power plants.

DATES: effective date: December 7, 1983.

The incorporation by reference of certain publications listed in the

regulations is approved by the Director of the Federal Register as of December 7, 1983.

FOR FURTHER INFORMATION CONTACT:

Ms. N. J. Miegel, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Telephone: (301) 443-7880.

SUPPLEMENTARY INFORMATION: On December 22, 1982 the Nuclear Regulatory Commission published in the Federal Register proposed amendments to its regulation, 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities." The proposed amendment would revise § 50.55a to incorporate by reference the Summer 1982 Addenda to Section III, Division 1, "Rules for the Construction of Nuclear Power Plant Components," of the ASME Boiler and Pressure Vessel Code.

The incorporation of the new Addenda does not change any of the previous supplementary requirements included in the regulation.

Some of the changes effected in the addenda that are incorporated by this rule follow:

1. The foreword of all sections of the Code was revised regarding interpretations of the Code to restrict the authority to issue such interpretations to the American Society of Mechanical Engineers. Previously, paragraph thirteen of the foreword had permitted the National Board of Boiler and Pressure Vessel Inspectors to issue interpretations of the ASME Boiler and Pressure Vessel Code.

2. Paragraph NCA-8230(b) was revised to delete the requirement that the location of the nameplate for component supports be shown on the support drawing.

3. Code Case N-100, "Pressure Relief Valve Design," was adopted into the body of the Code as paragraph NB-3590. The Code Case will be annulled when the Summer 1982 Addenda become effective.

4. Paragraph NB-5520 was revised to update the reference to the American Society for Nondestructive Testing (ASNT) standard SNT-TC-1A, from the 1975 to the 1980 Edition. It was also revised to clearly state that even if an outside agency or ASNT provided the qualification examinations, the employer is still responsible for certifying its own personnel.

Interested persons were invited to submit written comments for consideration in connection with the proposed amendment by February 22, 1983. One comment was received on the proposed rule. The Summer 1982 Addenda invoke the June 1980 Edition of Recommended Practice No. SNT-TC-

1A, "Personnel Qualification and Certification in Nondestructive Testing," in paragraph NB-5520 in lieu of the 1975 Edition of SNT-TC-1A which was previously invoked. The commenter objected to updating the edition of SNT-TC-1A referenced by the Code because the commenter was of the opinion that the 1980 Edition of SNT-TC-1A no longer requires Level III individuals to demonstrate their ability to perform the examinations for which they are being qualified. A detailed comparison of the 1975 and 1980 Editions of SNT-TC-1A reveals that although the requirements for qualifications of Level III individuals were changed, neither edition specifically requires a demonstration of the individual's ability to perform the examination. No changes to the rule were made in response to the comment.

The NRC staff is currently working with a committee of industry representatives that is developing an improved requirements document for the qualification and certification of nondestructive examination personnel. Additionally, the staff is involved in a study of the qualification and certification of various quality assurance and quality control personnel that includes consideration of nondestructive examination personnel. At this time, the staff has not concluded that hands-on practical examinations for Level III nondestructive examination personnel are warranted, and finds that the provisions of the June 1980 Edition of SNT-TC-1A are acceptable for Section III Code activities, pending the development of any new requirements.

In addition to the public comment, there was a concern raised by the NRC staff on the change in the 1980 Edition of SNT-TC-1A to the use of the word "should" in numerous places in the standard where the word "shall" had been used in the past. The concern centered around whether or not the change in language resulted in a change in the enforceability of the provisions of SNT-TC-1A. Because of the staff's concern, an inquiry, NI 83-033, was submitted to the American Society of Mechanical Engineers, Boiler and Pressure Vessel Code Committee, asking for an interpretation of the Code's endorsement of SNT-TC-1A. The response to the inquiry was that regardless of the language used in SNT-TC-1A, the Code's endorsement of SNT-TC-1A makes the provisions of SNT-TC-1A mandatory for Code activities.

Paperwork Reduction Act Statement

This final rule does not contain a new or amended information collection requirement subject to the Paperwork

Reduction Act of 1980 (44 U.S.C. 3501 et seq.). Existing requirements were approved by the Office of Management and Budget approval number 3150-0011.

Regulatory Flexibility Certification

In accordance with the Regulatory Flexibility Act of 1980, 5 U.S.C. 605(b), the Commission hereby certifies that this rule will not have a significant economic impact on a substantial number of small entities. This rule affects only the licensing and operation of nuclear power plants. The companies that own these plants do not fall within the scope of the definition of "small entities" set forth in the Regulatory Flexibility Act of the Small Business Size Standards set out in regulations issued by the Small Business Administration at 13 CFR Part 121. Since these companies are dominant in their service areas, this rule does not fall within the purview of the Act.

List of Subjects in 10 CFR Part 50

Antitrust, Classified information, Fire prevention, Incorporation by reference, Intergovernmental relations, Nuclear power plants and reactors, Penalty, Radiation protection, Reactor siting criteria, Reporting and recordkeeping requirements.

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974, as amended, and sections 552 and 553 of Title 5 of the United States Code, the following amendments to Title 10, Chapter 1, Code of Federal Regulations, Part 50 are published as a document subject to codification.

PART 50—DOMESTIC LICENSING OF PRODUCTION AND UTILIZATION FACILITIES

1. Authority: Secs. 103, 104, 161, 182, 183, 186, 189, 68 Stat. 936, 937, 948, 953, 954, 955, 956, as amended, sec. 234, 83 Stat. 1244, as amended (42 U.S.C. 2133, 2134, 2201, 2232, 2233, 2236, 2239, 2282); secs. 201, 202, 206, 88 Stat. 1242, 1244, 1246, as amended (42 U.S.C. 5841, 5842, 5846), unless otherwise noted.

Sec. 50.7 also issued under Pub. L. 95-601, Sec. 10, 92 Stat. 2951 (42 U.S.C. 5851). Secs. 50.58, 50.91 and 50.92 also issued under Pub. L. 97-415, 96 Stat. 2073 (42 U.S.C. 2239). Sec. 50.78 also issued under sec. 122, 68 Stat. 939 (43 U.S.C. 2152). Secs. 50.80-50.81 also issued under sec. 184, 68 Stat. 954, as amended (42 U.S.C. 2234). Secs. 50.100-50.102 also issued under sec. 186, 68 Stat. 955 (42 U.S.C. 2236).

For the purposes of sec. 223, 68 Stat. 958, as amended (42 U.S.C. 2273), §§ 50.10 (a), (b), and (c), 50.44, 50.46, 50.48, 50.54, and 50.80(a) are issued under sec. 161b, 68 Stat. 948, as amended (42 U.S.C. 2201(b)); §§ 50.10 (b) and (c) and 50.54 are issued under sec. 161i, 68 Stat. 949, as amended (42 U.S.C. 2201(i)); and §§ 50.55(e), 50.59(b), 50.70, 50.71, 50.72, and

50.78 are issued under sec. 161o, 68 Stat. 950, as amended (42 U.S.C. 2201(o)).

2. In § 50.55a, paragraph (b)(1) is revised to read as follows:

§ 50.55 Codes and standards.

(b) * * *

(1) As used in this section, references to Section III of the ASME Boiler and Pressure Vessel Code refer to Section III, Division 1, and include editions through the 1980 Edition and addenda through the Summer 1982 Addenda.

Dated at Bethesda, Maryland, this 5th day of October 1983.

For the Nuclear Regulatory Commission.

Jack W. Roe,

Acting Executive Director for Operations.

[FR Doc. 83-30001 Filed 11-3-83; 8:45 am]

BILLING CODE 7590-01-M

DEPARTMENT OF TRANSPORTATION

Federal Aviation Administration

14 CFR Part 39

[Docket No. 83-CE-75-AD; Amdt. 39-4763]

Airworthiness Directives; Cessna Models, 152, A152, 172N, 172P, 172Q, 172RG, R172K, 180K, 182Q, 182R, T182, R182, TR182, A185F, F152, FA152, F172N, F172P, FR172K, F182Q, and FR182 Airplanes.

AGENCY: Federal Aviation Administration (FAA), DOT.

ACTION: Final rule.

SUMMARY: This amendment adopts a new Airworthiness Directive (AD), applicable to certain Cessna Models, 152, A152, 172N, 172P, 172Q, 172RG, R172K, 180K, 182Q, 182R, T182, R182, TR182, A185F, F152, FA152, F172N, F172P, FR172K, F182Q, and FR182 airplanes. It requires a one time inspection of the aileron hinges for correct location of the cotter pins which retain the aileron hinge pins and if necessary, replacement or correction of aileron hinges having incorrectly located cotter pins. It has been determined that the hole for the cotter pin which retains the aileron hinge pin may have been mislocated on some aileron hinges. This may result in loss of the aileron hinge pin and aileron control. The inspection and corrective action will preclude loss of the hinge pin.

DATES: Effective date: November 8, 1983.

Compliance: As prescribed in the body of this AD.

ADDRESSES: Copies of Cessna Service Information Letter SE83-18 dated July 29, 1983, applicable to this AD may be

obtained from Cessna Aircraft Company, P.O. Box 1521, Wichita, Kansas 67201, Telephone No. (316) 685-9111. A copy of this information is also contained in the Rules Docket, FAA, Office of the Regional Counsel, Room 1558, 601 East 12th Street, Kansas City, Missouri 64106.

FOR FURTHER INFORMATION CONTACT:

Douglas W. Haig, Aerospace Engineer, Wichita Aircraft Certification Office, Room 238, Terminal Building 2299, Mid-Continent Airport, Wichita, Kansas 67209, Telephone No. (316) 269-7005.

SUPPLEMENTARY INFORMATION: A

Cessna Model TR182 airplane experienced an in-flight aileron control problem. Upon landing it was found that the outboard hinge pin on the right aileron was missing. The normal airloads, coupled with the loss of the hinge pin caused the right aileron to bend upward, temporarily jamming the controls. The pilot was able to stop the resulting right roll by applying left rudder. The airplane assumed a nose down attitude although full nose up trim was applied. Subsequently the aileron broke loose and the airplane was landed without further incident.

A hole is drilled on each end of this piano style hinge through which a cotter pin is installed to retain the hinge pin. On the airplane involved in this incident the cotter pin holes were mislocated and the installed cotter pins did not retain the hinge pin in the hinge.

Investigation into the manufacturing of the aileron hinge revealed that it was possible to misalign the hinge piece in the tool used to locate and drill the cotter pin hole. Inspection failed to find the discrepant parts due to the sampling techniques utilized. The manufacturer has, however, isolated the time period during which this specific tooling was used and in-turn, the airplanes potentially affected. Cessna has issued Service Information Letter SE83-18 dated July 29, 1983, requiring a one time inspection, plus corrective action if necessary, on the affected airplanes.

Since the FAA has determined that the unsafe condition described herein is likely to exist or develop in other airplanes of the same type design, an AD is being issued requiring a one time visual inspection and replacement or correction of aileron hinges with mislocated cotter pin holes in accordance with Cessna Service Information Letter SE83-18 dated July 29, 1983, on certain Cessna Models, 152, A152, 172N, 172P, 172Q, 172RG, R172K, 180K, 182Q, 182R, T182, R182, TR182, A185F, F152, FA152, F172N, F172P, FR172K, F180Q, and FR182 airplanes.